

# SPARC



## SPARC and the High Magnetic Field Path to Fusion Energy

Presented by Martin Green  
*MIT – Plasma Science & Fusion Center*


LLNL HEDSC Seminar – March 25, 2025

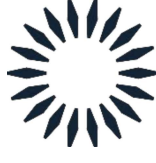
# Acknowledgements – The SPARC Team, Particularly

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– Sean Ballinger	– Jerry Hughes	– Brandon Sorbom*	Collaborating Ins
– Dan Brunner*	– Adam Kuang	– Ryan Sweeney	– Aalto U.
– Alex Creely*	– Yijun Lin	– Alex Tinguely	– Chalmers
– Chris Chrobak*	– Earl Marmor	– Libby Tolman	– Columbia U.
– Darren Garnier	– Bob Mumgaard*	– Dennis Whyte	– Fiat Lux
– Bob Granetz	– Matt Reinke*	– John Wright	– GA
– Zach Hartwig	– Pablo Rodriguez	– Steve Wukitch	– ORNL
– Nathan Howard	– Steve Scott*	– SPARC Team	– PPPL
			– UCSD
			– U. York

\*Commonwealth Fusion Systems

**PSFC** 



**Commonwealth Fusion**



**SPARC**

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# Outline

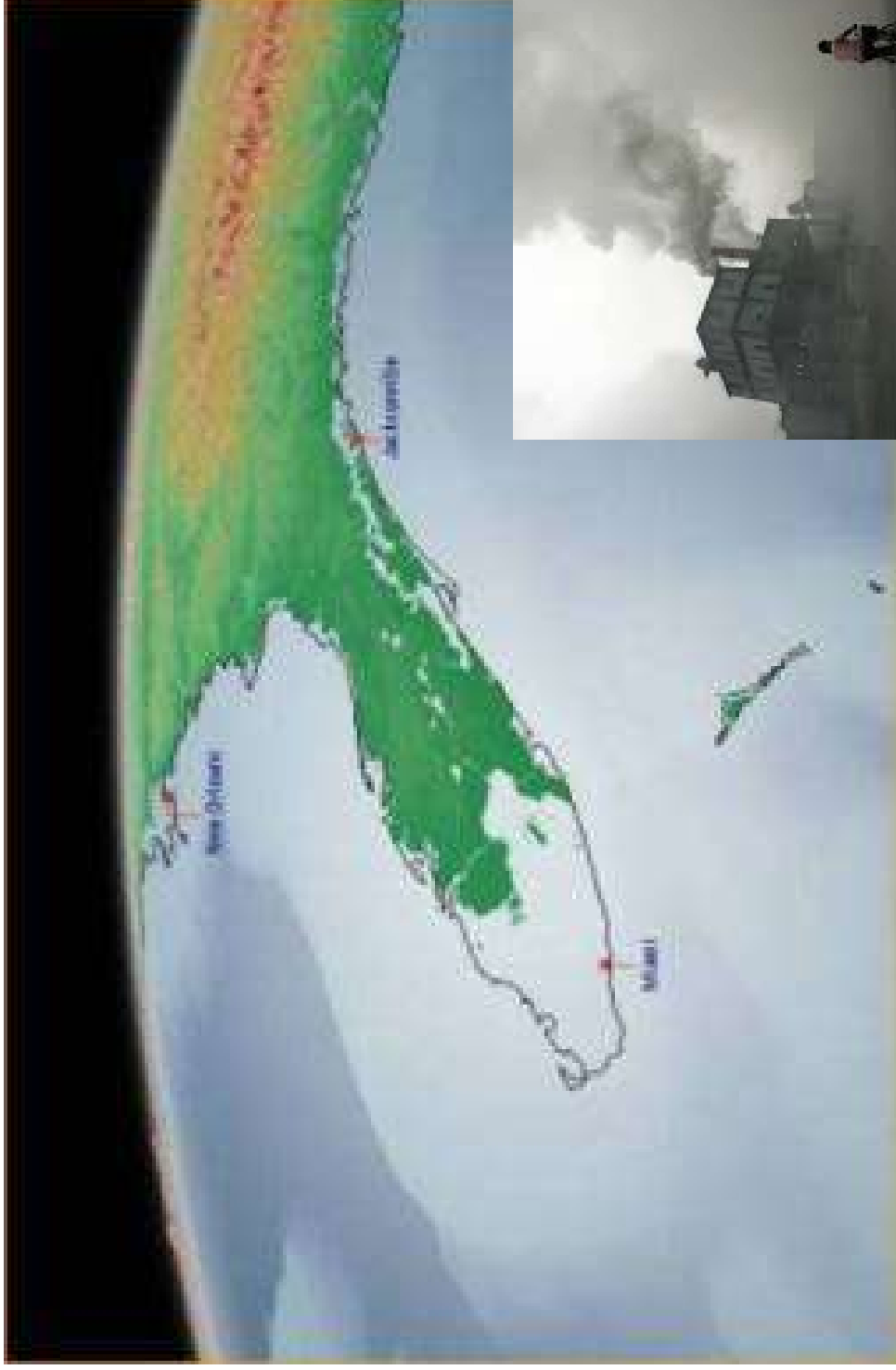
- Quick introduction to fusion energy and magnetic confinement
- High Temperature Superconductors – a game-changing technology
- Plans, projections and physics for the SPARC device
- The role of private industry



SPARC – planned as the first magnetic confinement which would make net energy



# The World Needs Reliable, Safe, Clean, Carbon-Free Energy



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# The World Needs Reliable, Safe, Clean, Carbon-Free Energy



**Can Fusion Contribute To The Solution?**



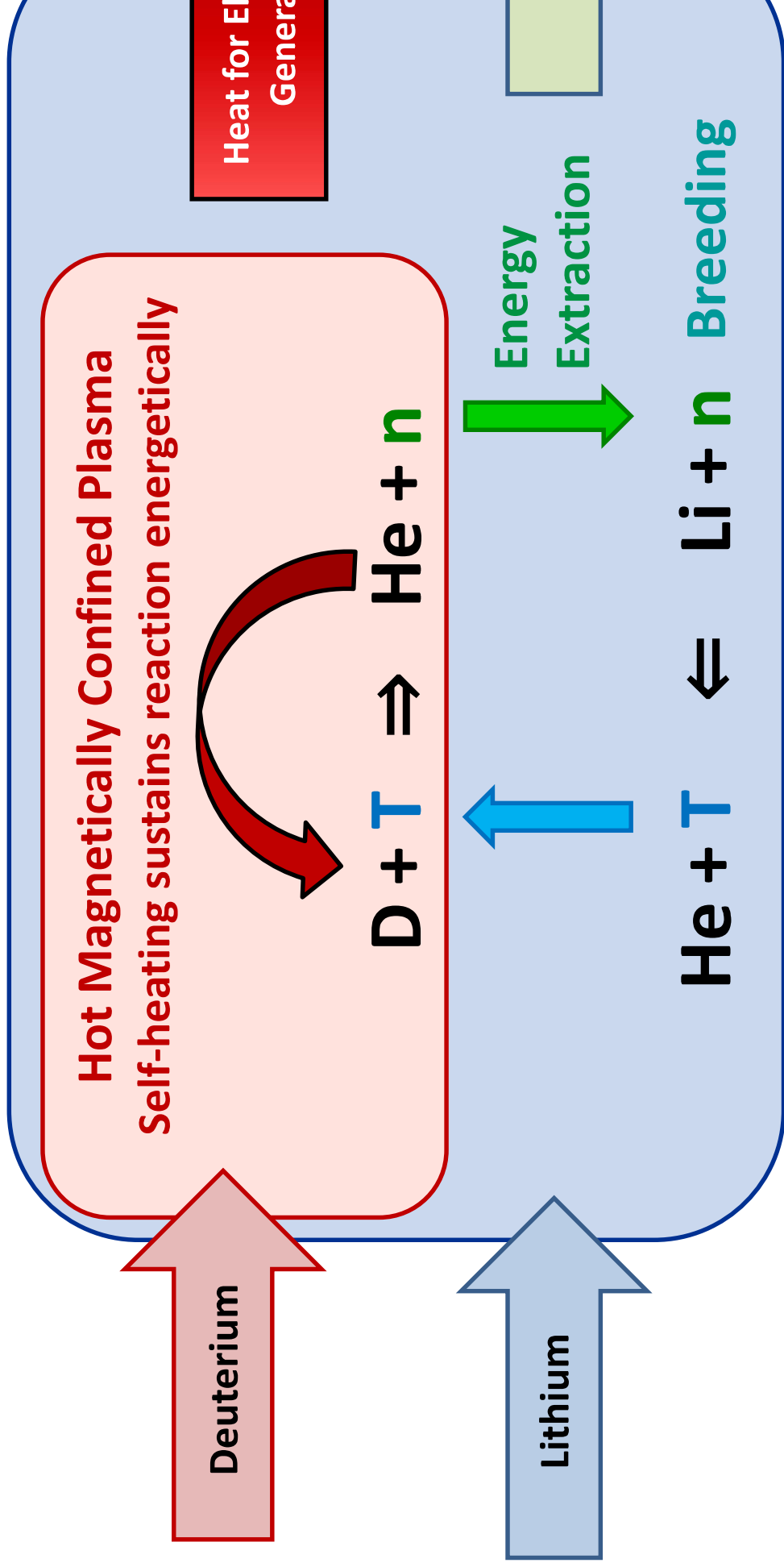
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# The Technical Challenges Are Well Understood

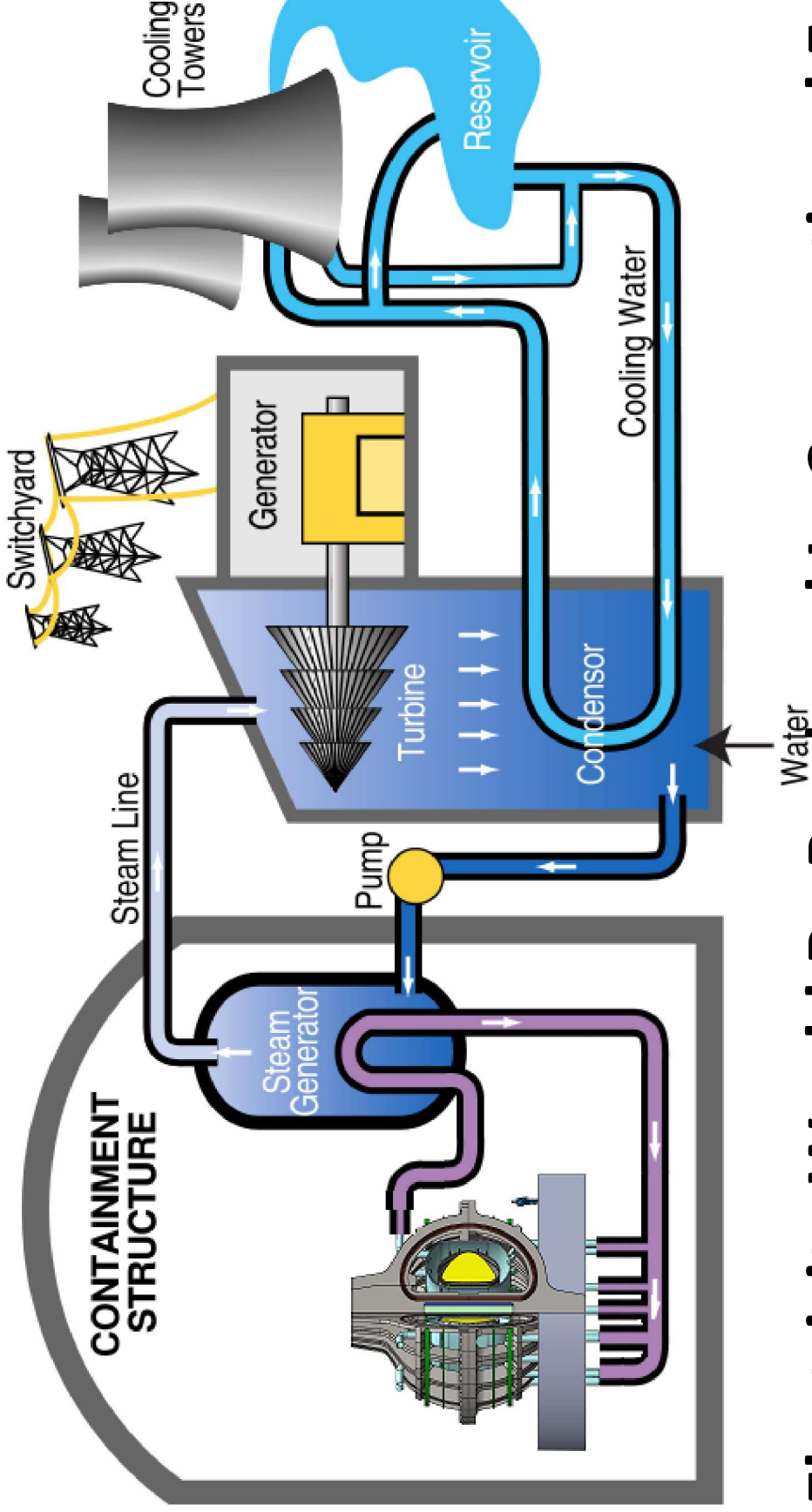
- 1. Plasma physics**
  - Create, confine and sustain hot plasmas that produce net energy
- 2. Taming the plasma material interface**
  - Minimize heat and particle loads
  - Develop materials and strategies to handle what remains
- 3. Harnessing fusion energy**
  - Fuel cycle – tritium breeding, inventory control
  - Structural materials – maintaining structural, thermal and electrical properties under intense neutron bombardment
  - Reliability, Availability, Maintainability, Inspectability





# Beyond the Fusion Core

## Electricity Would Be Produced In Conventional Fashion



## Electricity Would Be Produced In Conventional Fashion



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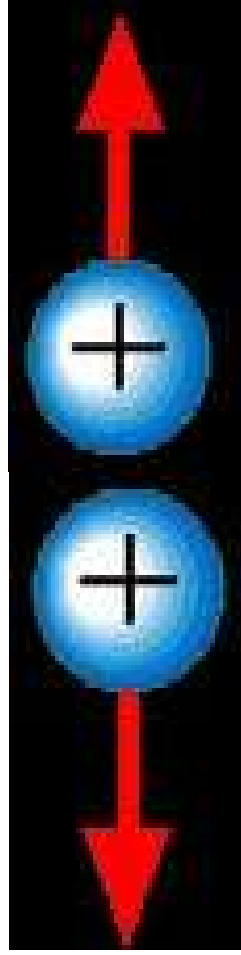


# The Physics Challenge:

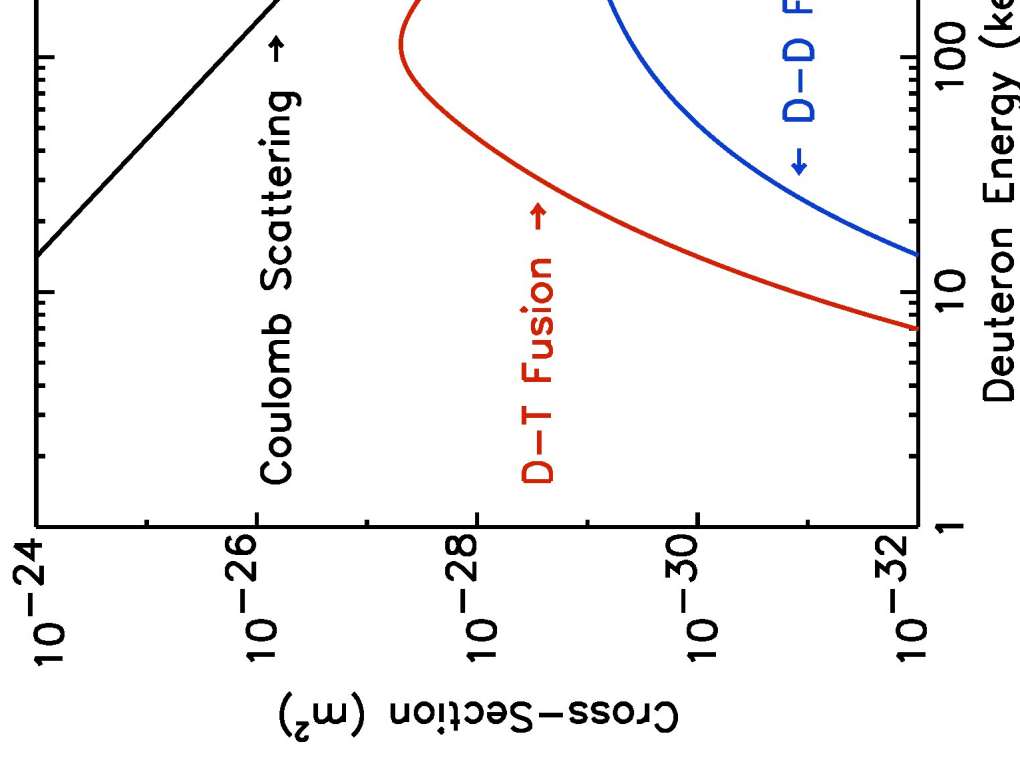
## Very High Kinetic Energies Are Needed To Overcome Electrostatic

Fusion is mediated by the strong nuclear force  
– with a range of  $10^{-15}$  m

So we need significant wave function overlap at  
that range



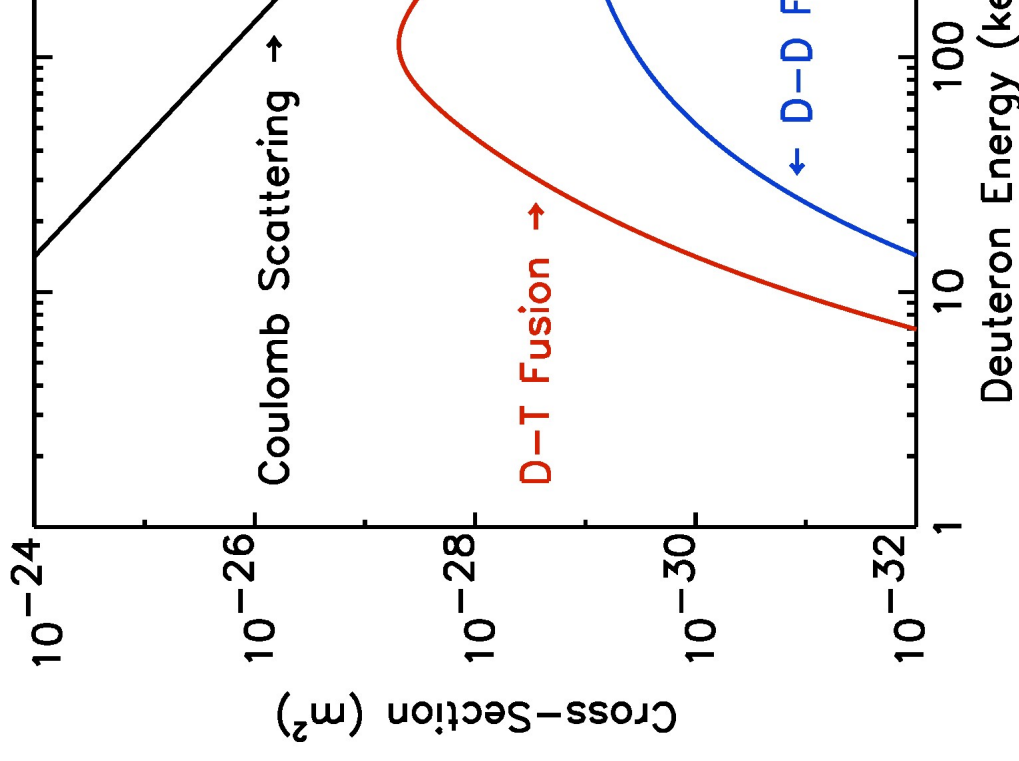
Nuclei are charged  $\Rightarrow$  Fusion requires ion  
energies on the order of 100 keV



## The Physics Challenge:

# Very High Kinetic Energies Are Needed To Overcome Electrostatic

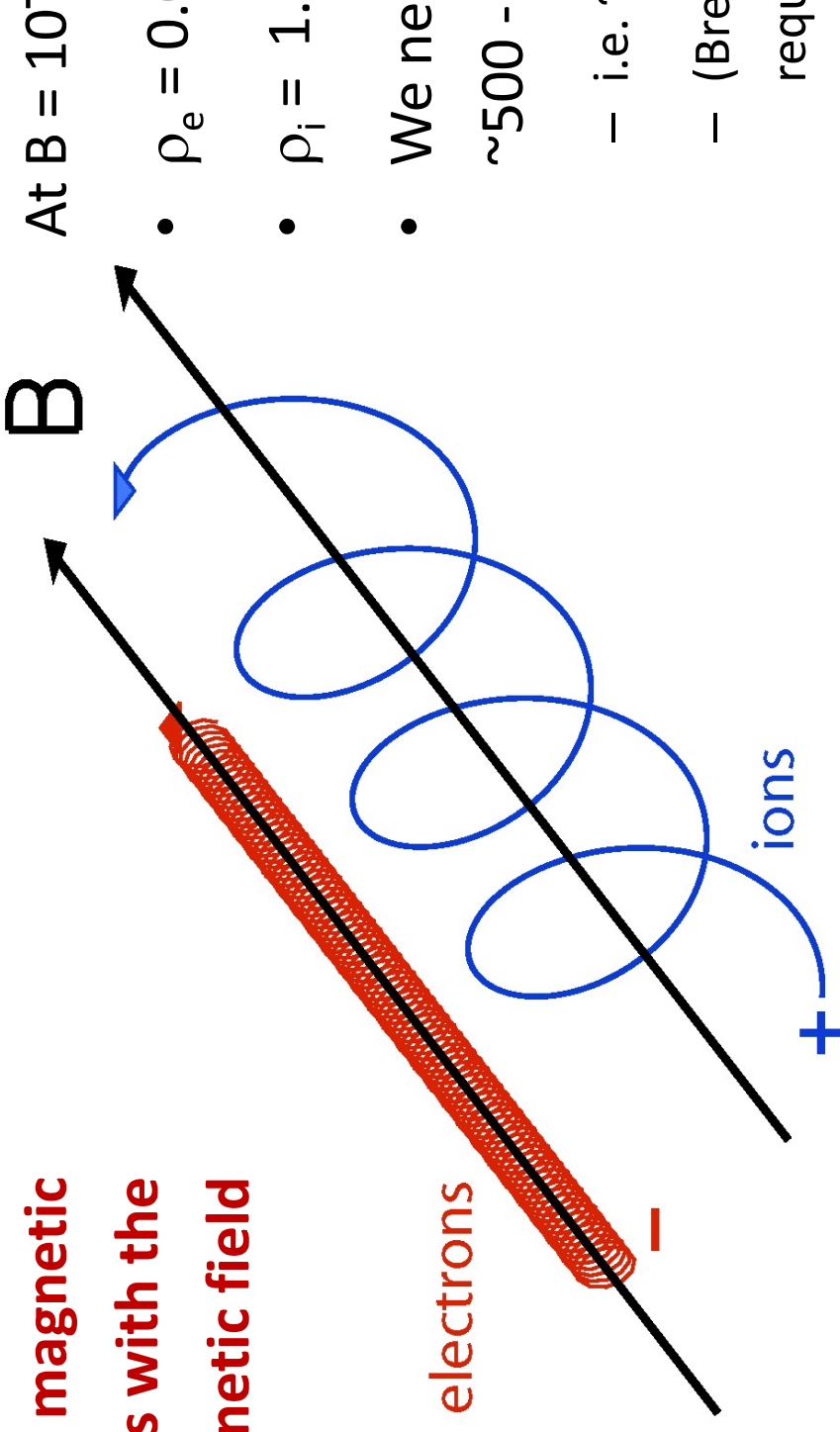
- But, even at the optimum energy, the nuclei are much more likely to scatter elastically than to fuse
- Particles rapidly thermalize, equilibrate
- We need to confine hot plasmas for many collision times
- Fusion plasmas need to reach temperatures on the order of 20 keV  $\approx$  200,000,000 K
- Must be isolated and insulated from ordinary matter



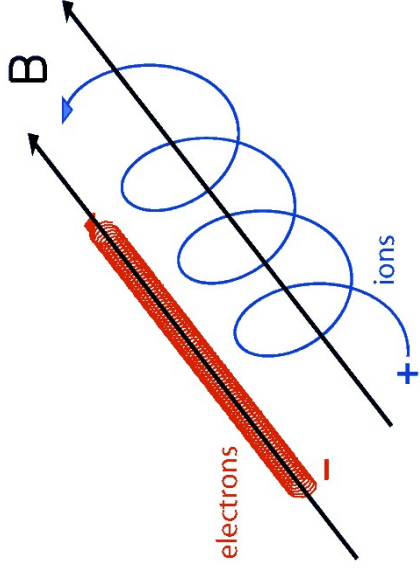
# Plasma Can Be Confined by the Gyration Of Charged Particles in a Magnetic Field

## Magnetic Confinement = Thermal Insulation Perpendicular To Field

The quality of the magnetic insulation increases with the strength of the magnetic field



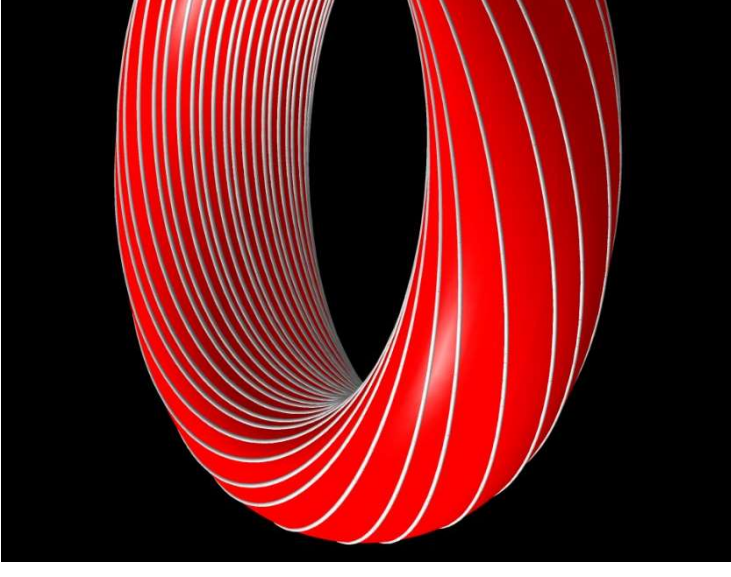
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# Magnetic fields confine particles in the perpendicular direction

- At the temperatures involved, ions are moving at over 1,000 km/s
- But, even at these temperatures, the average time for an ion to fuse  $\sim$  25 seconds
- For a practical device, the end losses must be eliminated

**Voila!**  
**Eliminate the ends**



A torus is unique topologically. It is the only 3D shape where the surface and the vector field (B in our case) can be tangent to the surface.

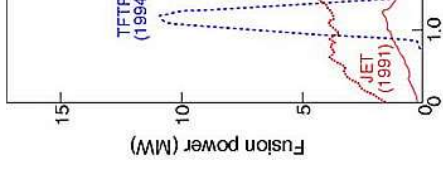
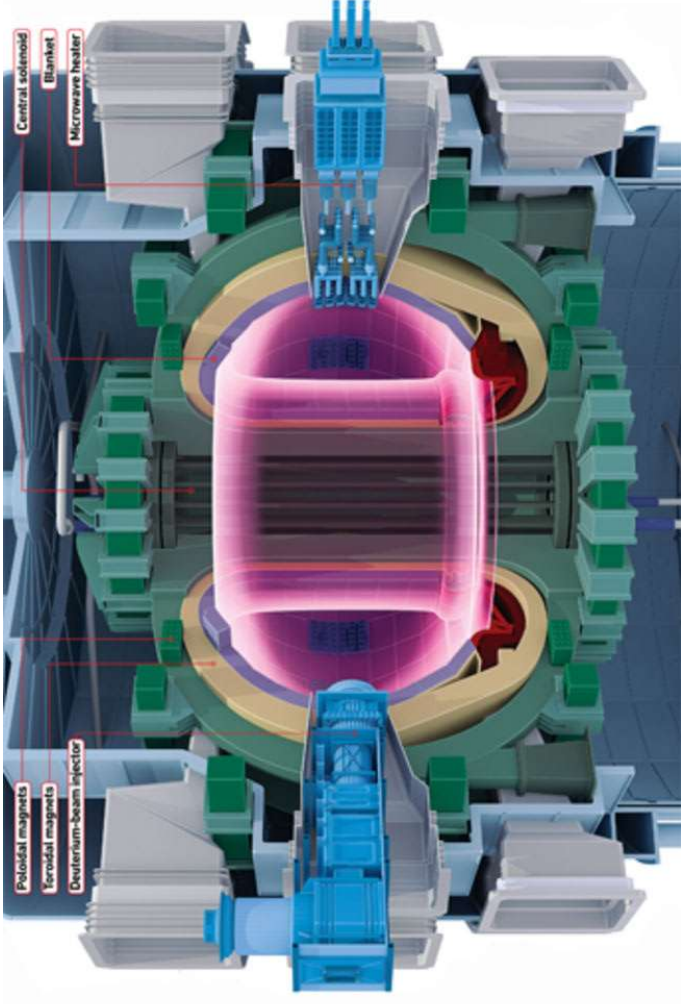


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# The Tokamak is the Leading Concept in the Race for Practical Fusion Energy

Nature provides a well-founded and quantitative set of metrics for progress in fusion energy in the Lawson criteria (1955):  $n\tau_E$  and  $T_i$  (Density x confinement time and temperature)

We've achieved the required  $n\tau_E$  and  $T_i$  individually, but not both together - YET



Tokamak  
produces  
MW of



# Tokamaks Demonstrated Enormous Progress For 3 Decades, But



HT-7 (CN)



JT-60U (JP)



KSTAR (KR)



MAST-U (UK)



NSTX-U (US)



Tore Supra (FR)



Alcator C-Mod (US)



ASDEX-U (DE)



COMPASS (CZ)



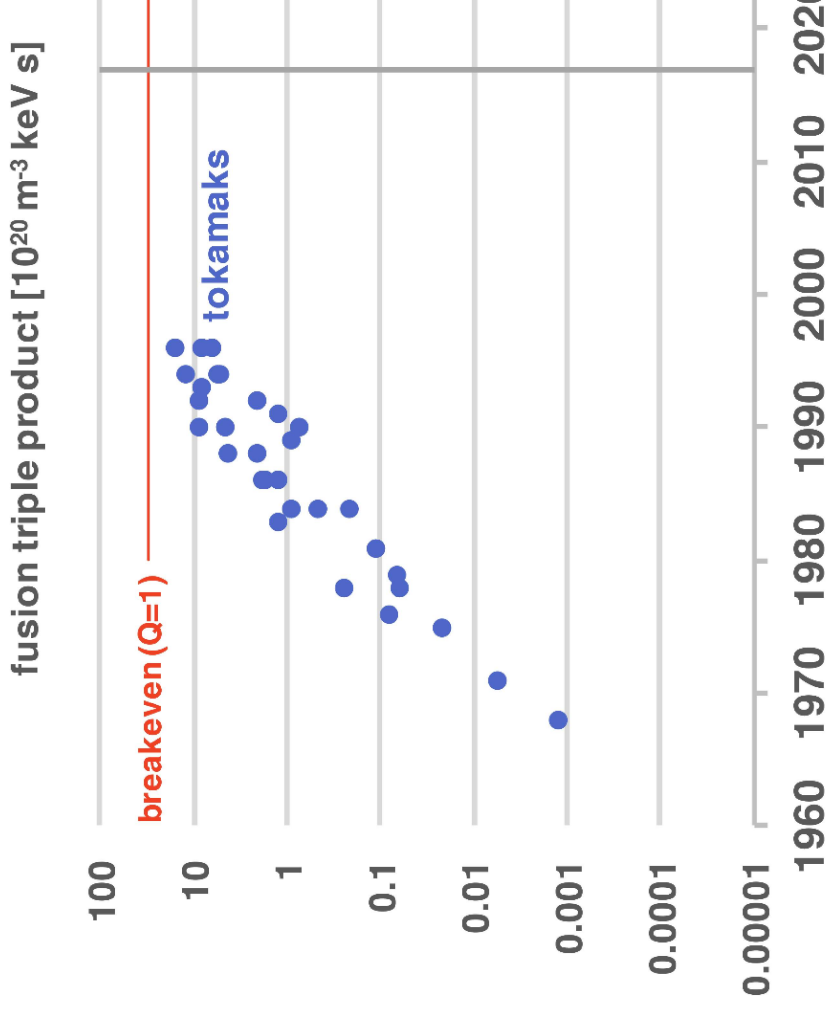
DIII-D (US)



EAST (CN)



FTU (IT)



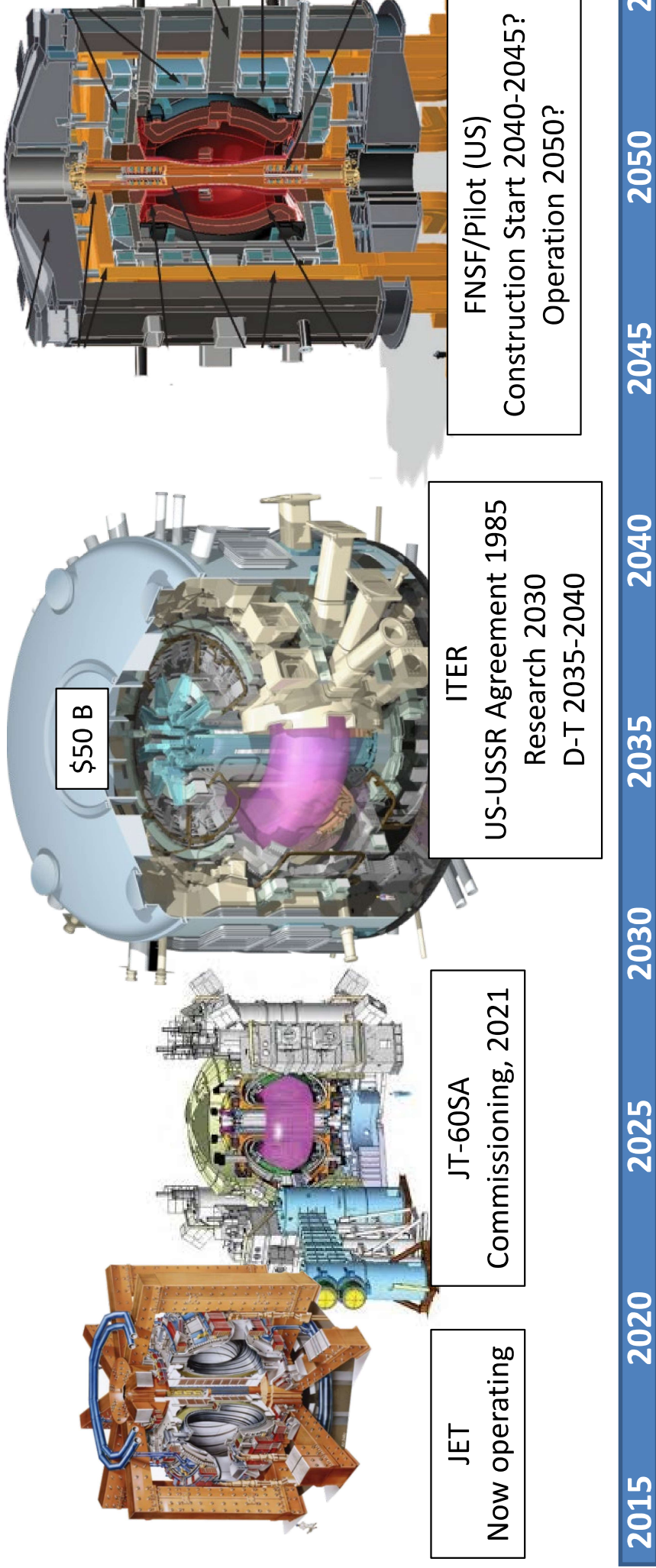
+ 160 other tokamaks across 60 years

Enormous technical and scientific base

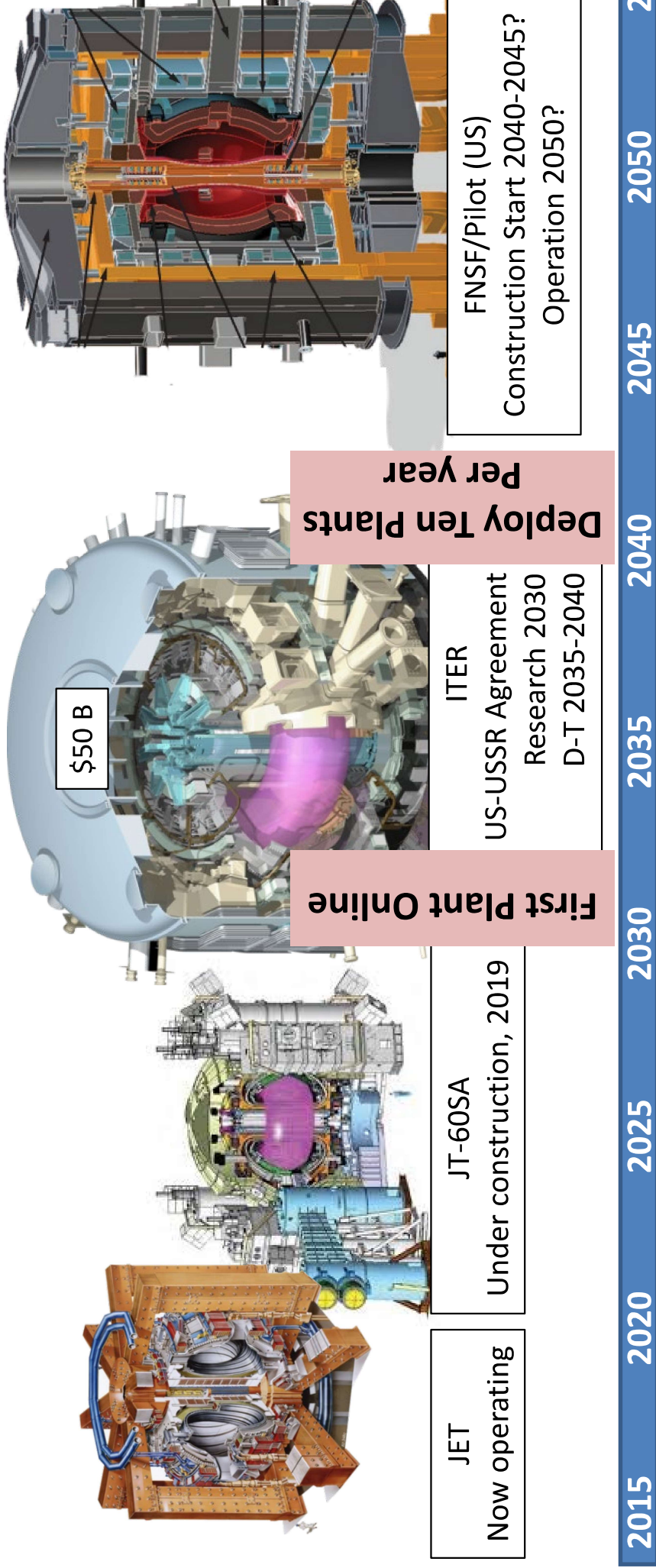
Progress Exceeded Moore's Law



# Progress Slowed As The Devices Got Too Big, Too Expensive

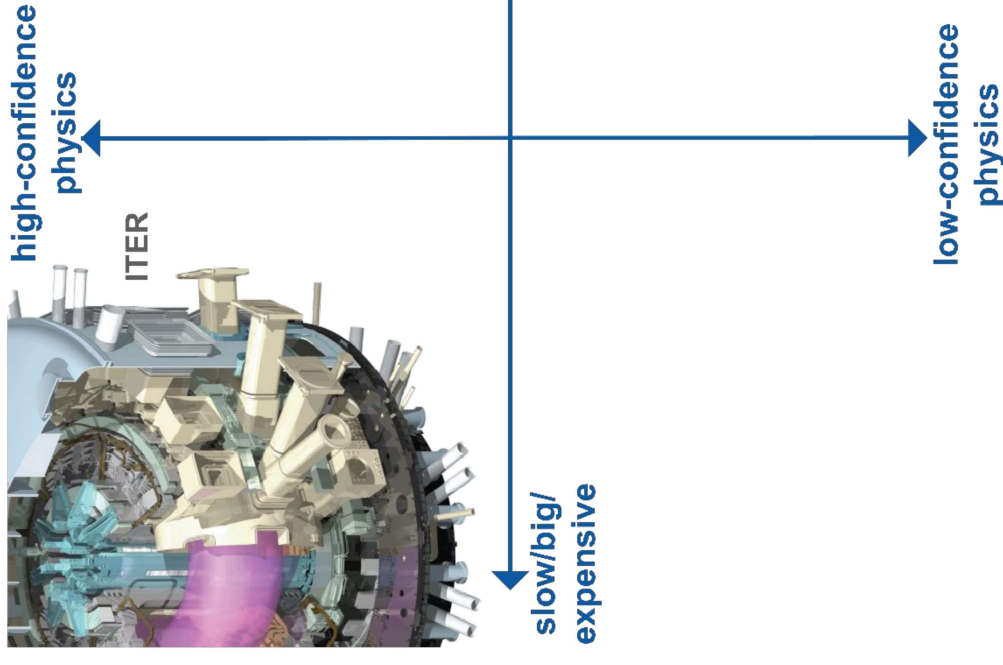
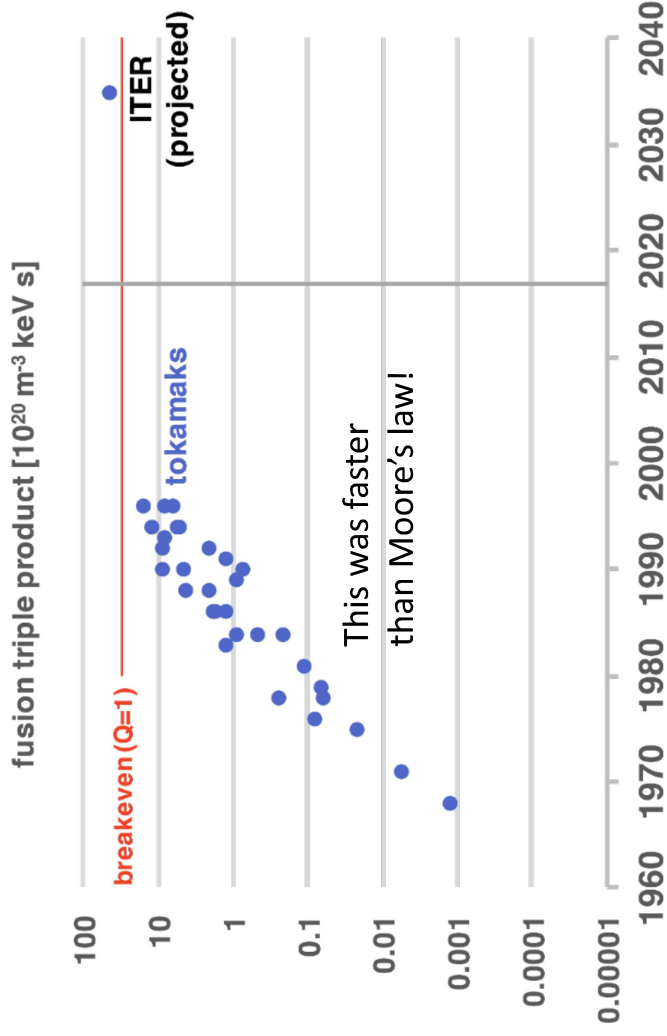


# Progress Slowed As The Devices Got Too Big, Too Expensive





# What Can We Do About This?

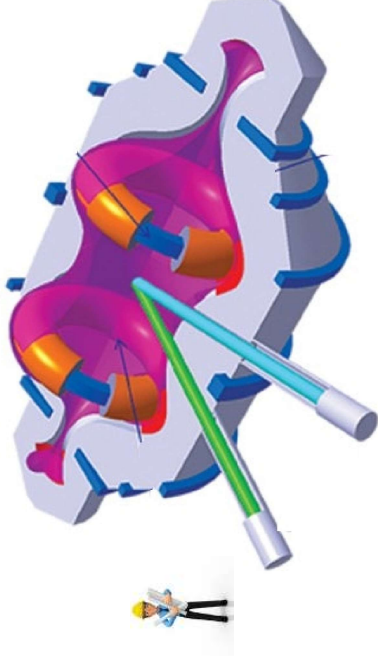
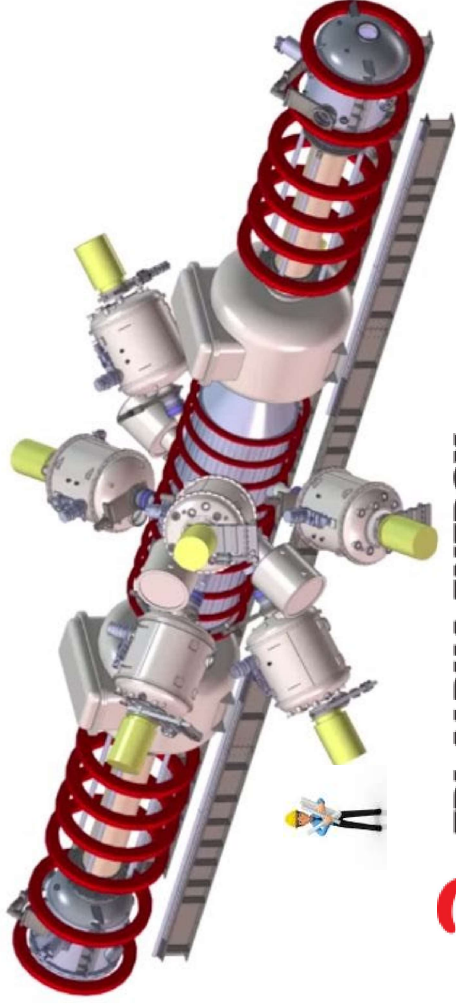


- Tokamaks: Impressive and unparalleled fusion performance
- Stagnating due to size & cost, not saturation due to physics
- Can they be fielded in time to deal with global warming?

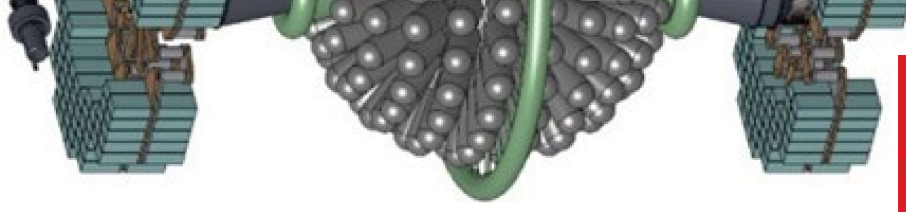


# Private Companies Have Attracted \$\$\$ On Small But Risky Co

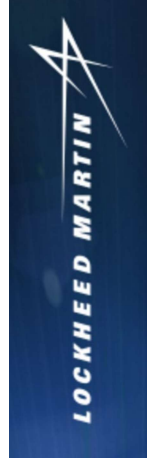
- Small enough to carry out without government funding
- Can be built quickly
- Raise the profile of fusion
- Have been innovative and nimble
- But – are taking a big leap into unknown physics and engineering



(all figures to scale)

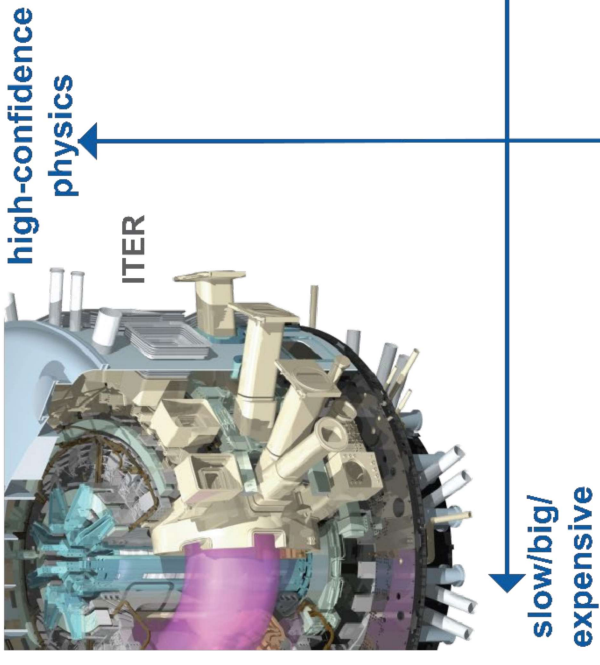
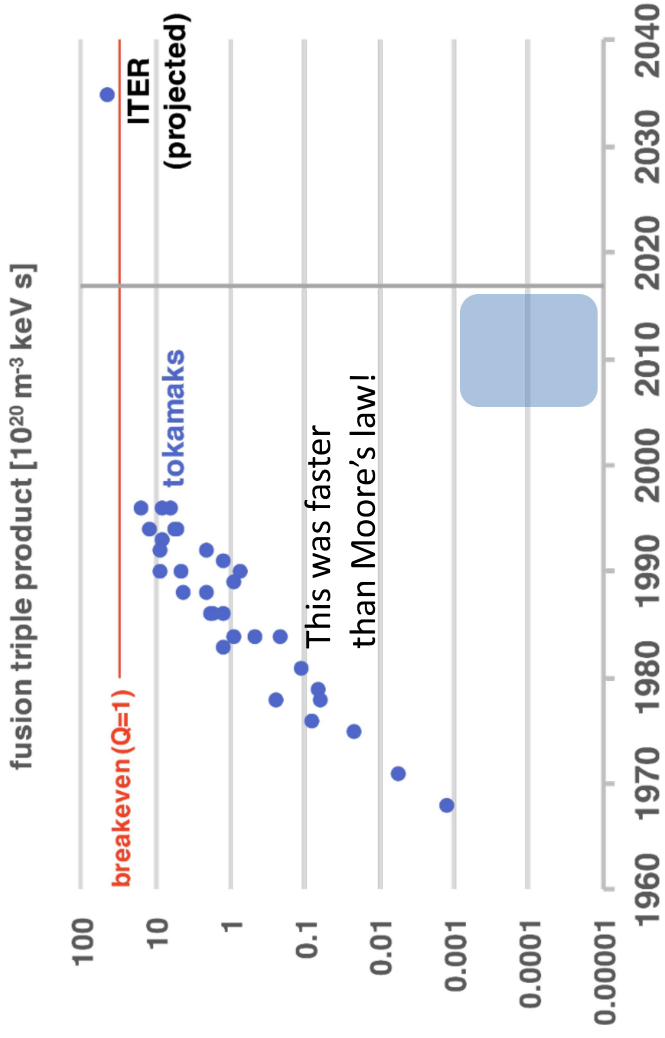


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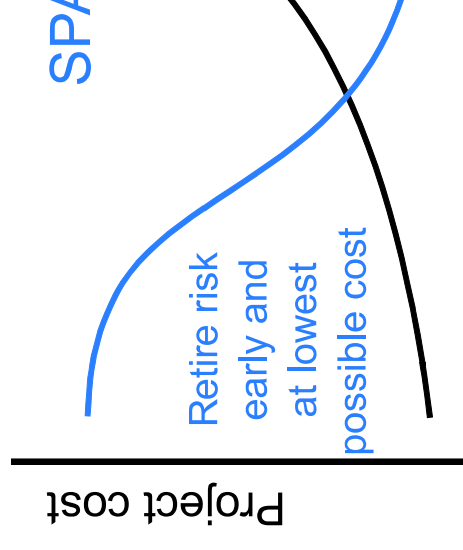
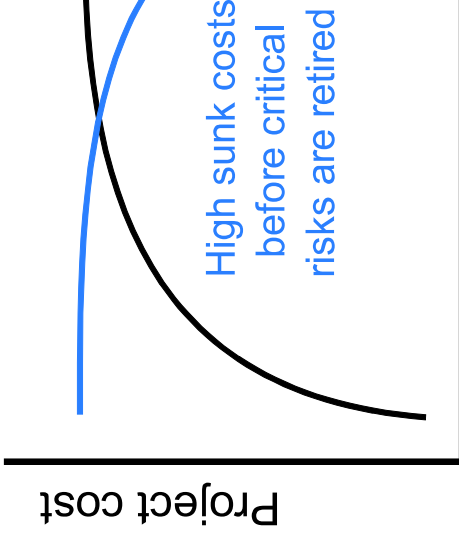
# These Alternates Currently Fall Far Short of What's Needed



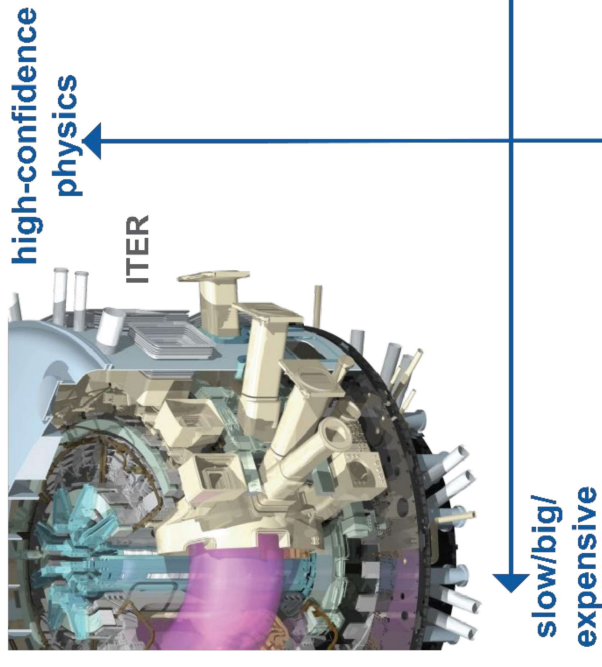
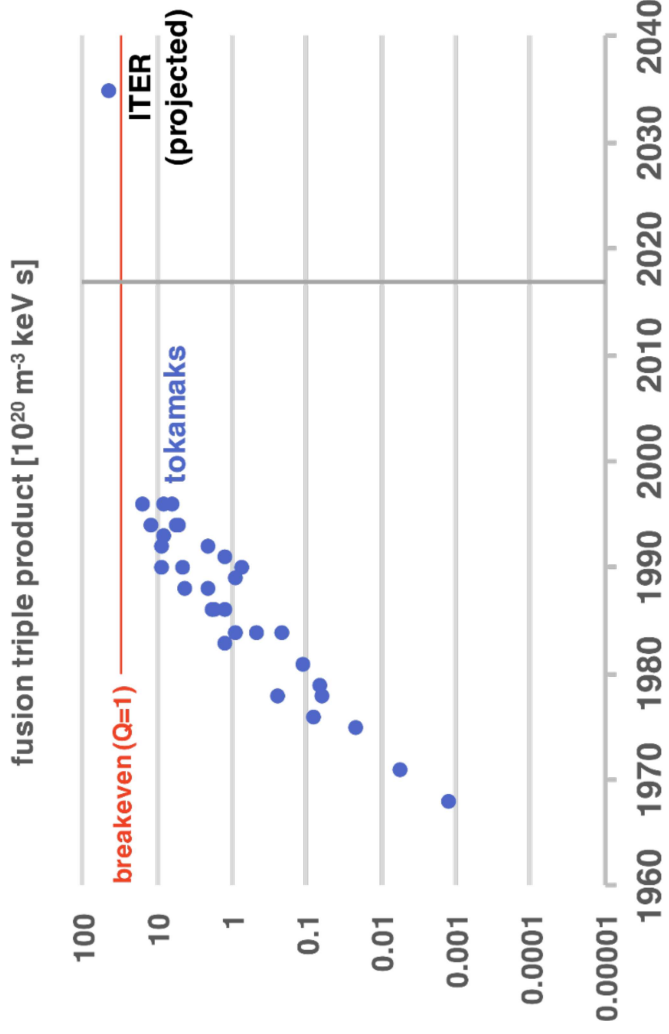
# Moreover Can We Follow a Development Path That Retires Ris

What should the risk retirement and cost curves look like?

- Retire the largest portion of the risk at lowest expenditure
- If the risk is in the physics then the entire device must be built before that risk is retired
- But if the risk is in the enabling technology, that can be retired early leaving high confidence the device will operate as designed
- Innovate in the areas of highest return
- Avoid putting all innovation on unproven physics and instead innovate in engineering of new components to make proven physics work better



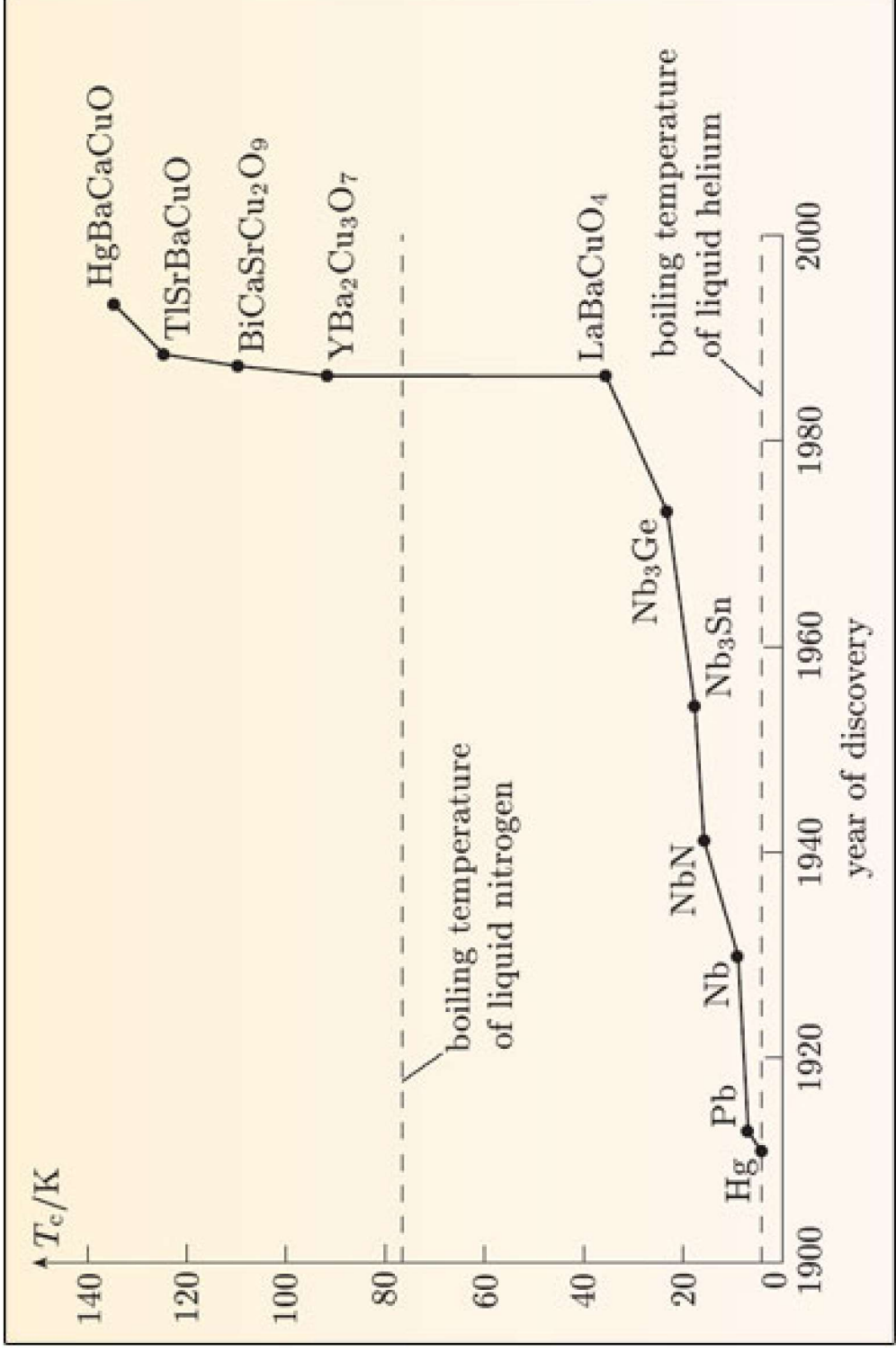
# We Need To Consider Another Approach



- We're looking for a path which builds on the progress and knowledge built by decades of research on the tokamak
- Reduced in scale to learn fast
- Focused on fusion power plants that are practical and economical



# We Think The Basis For Breakthrough Is Here – High Temperature Superconductors



- Discovered more than 100 years ago – 1911
- Nobel prize for Heike Kamerlingh Onnes and Pieter H. Heerike Muller 1911
- High  $T_c$  was a surprise – BCS theory still incomplete
- Ceramic form – brittle
- Too brittle for practical applications by 1980s



# Some People Understood The Implications Right Away

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*Journal of Fusion Energy, Vol. 7, No. 1, 1988*

## **Tokamak Reactor Concepts Using High Temperature High-Field Superconductors<sup>1</sup>**

**D. R. Cohn,<sup>2</sup> J. Schwartz,<sup>2,3</sup> L. Bromberg,<sup>2</sup> and J. E. C. Williams<sup>2</sup>**

<sup>2</sup>Plasma Fusion Center, Massachusetts Institute of Technology,  
Cambridge, Massachusetts 02139 U.S.A.

*Received April 4,*

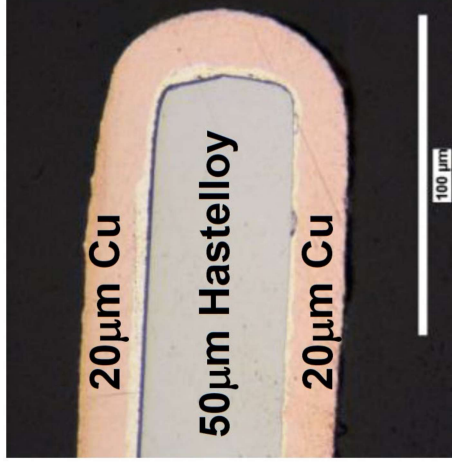
The rest of us took a bit more time to appreciate the implications



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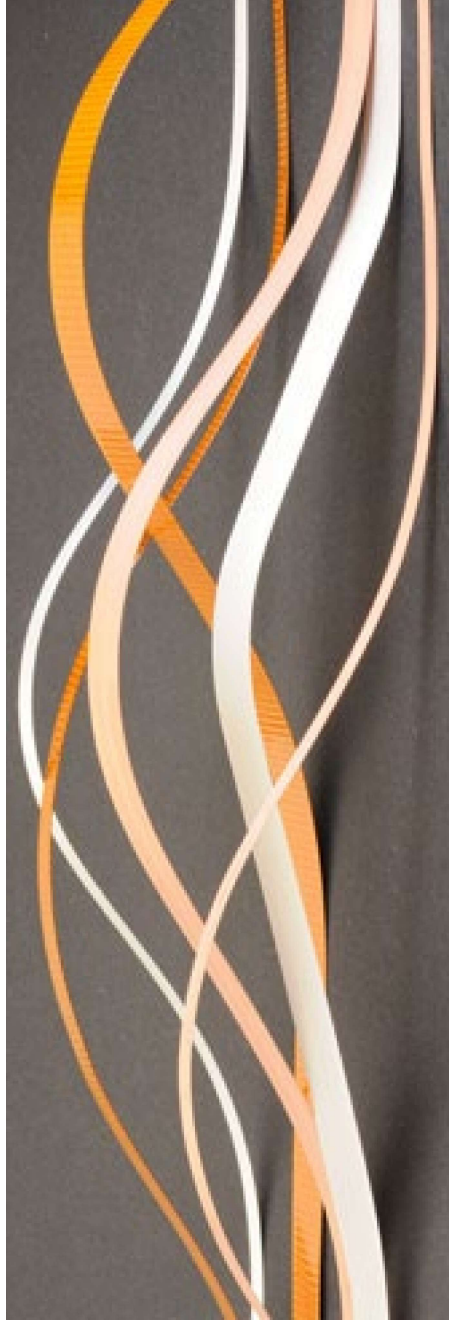
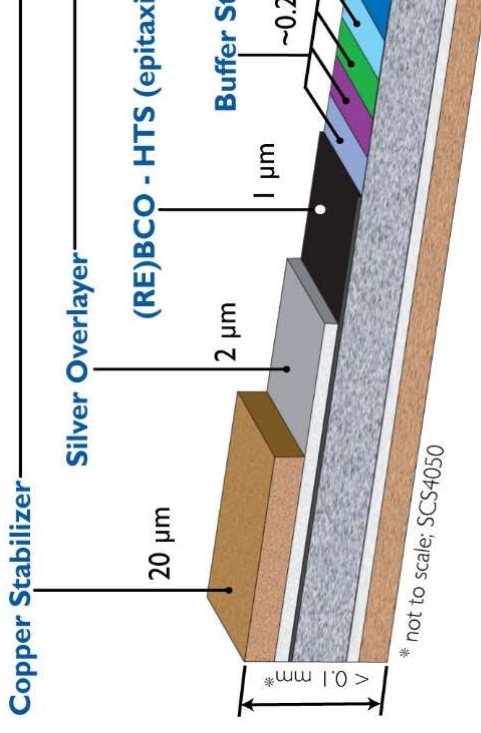
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# High Temperature Superconductors Out of the Lab and into Industrial Production



> 0.1 mm

- Deposited as a thin film on a steel substrate
- Over-coated with copper for stability

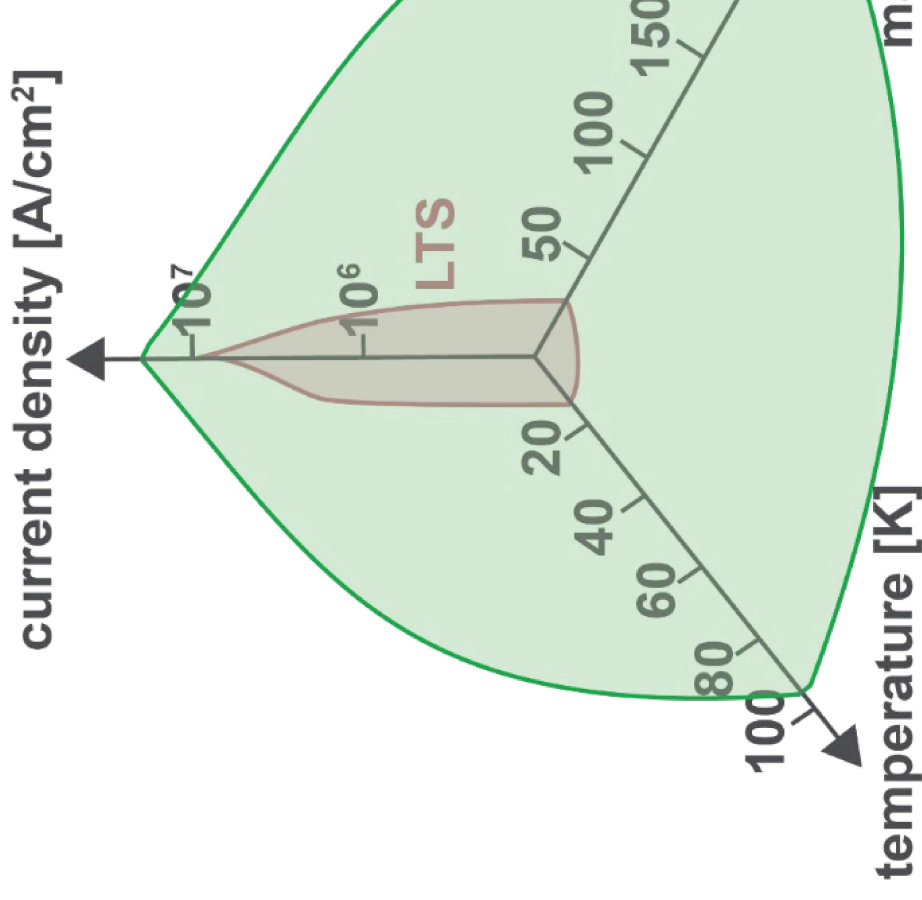




# These New Superconducting Materials Are Ideal For Large-Voltage Fusion Magnets

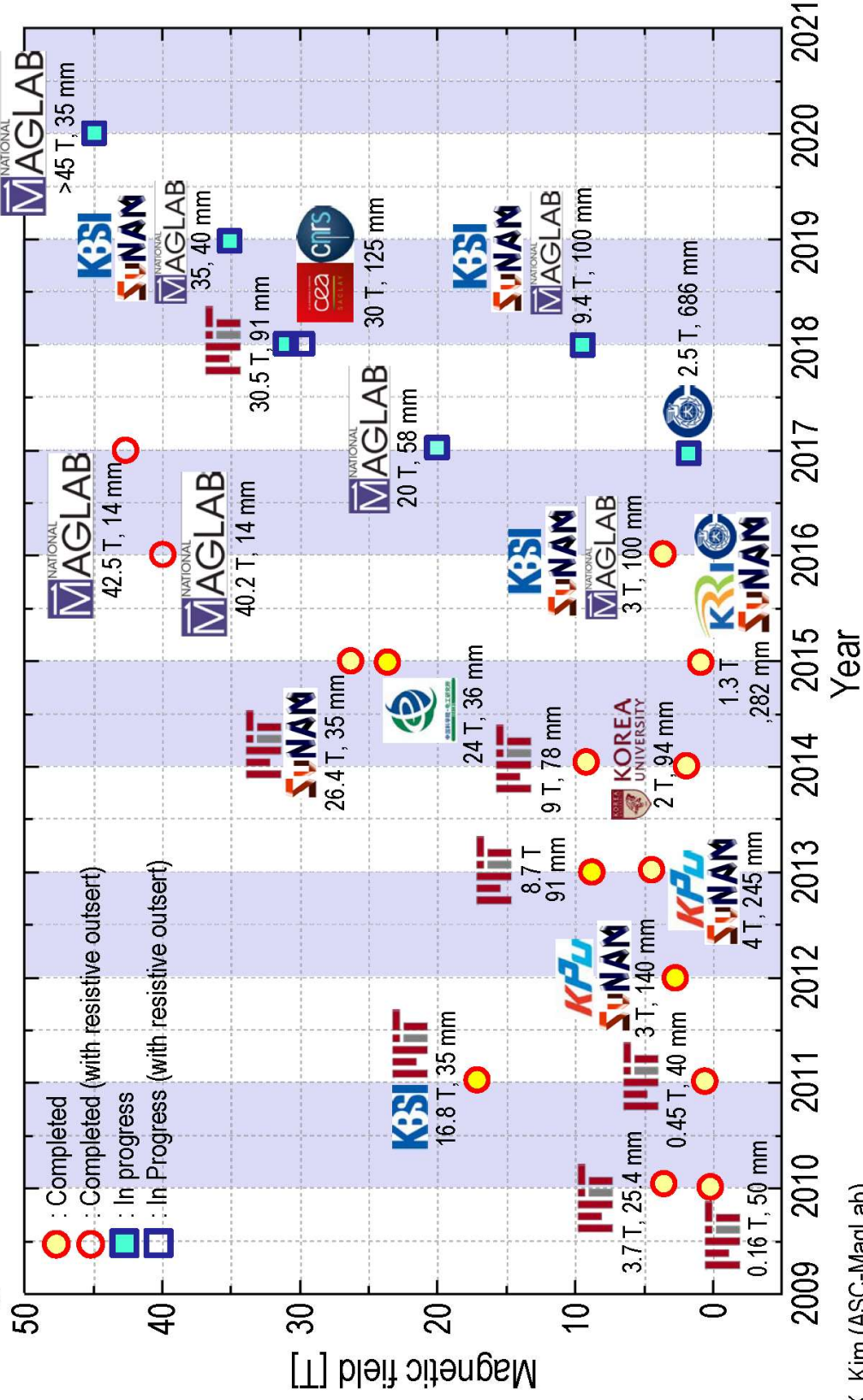
## High-Field Fusion Magnets

- Much wider operating space in field, current density and temperature
- Higher current densities – more compact magnets
  - Strong structure - substrate is mainly steel
- Operation at higher temperatures
  - New cryogenic options
  - Better stability against quench
  - Higher heat capacity should enable magnets with demountable joints
- No reaction process as part of winding



# Capabilities of HTS Magnets Are Rapidly Increasing, Costs Decreasing

## ■ Magnetic Field vs. Year



26 T



S. Hahn  
NHMFL, I

K. Kim (ASC-MagLab)



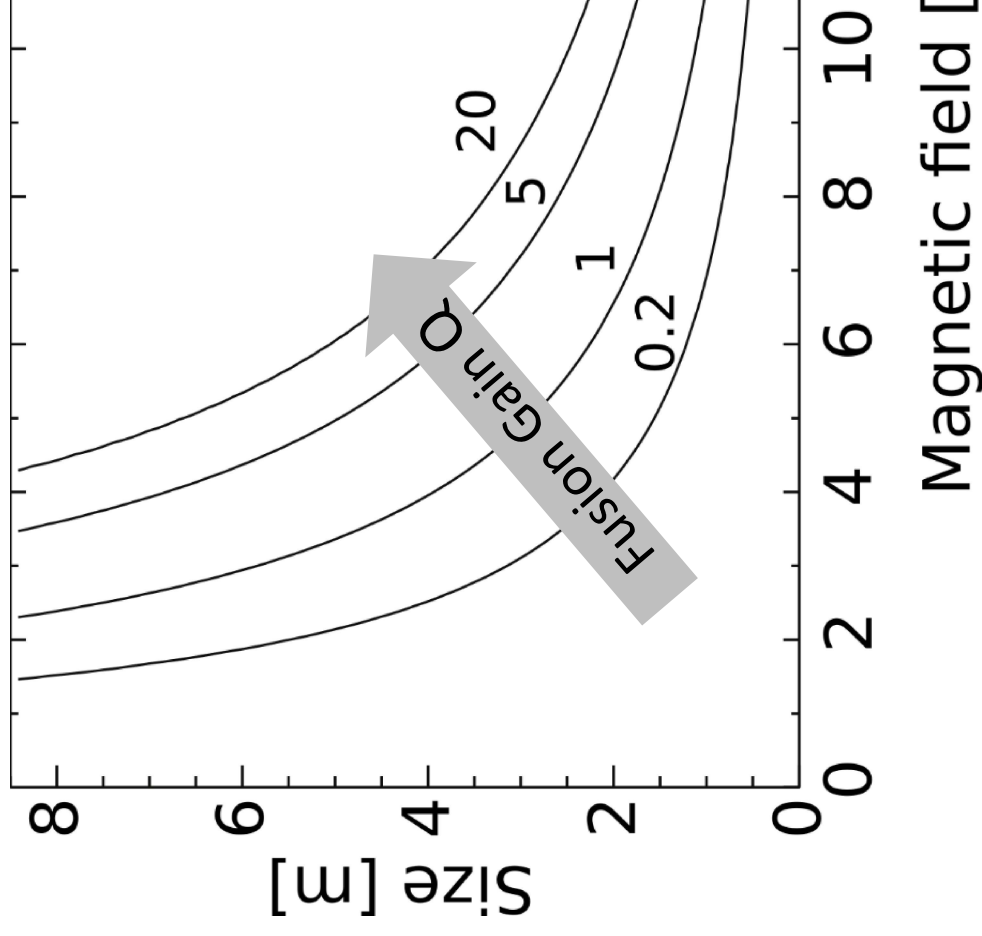
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# Why Do We Care?

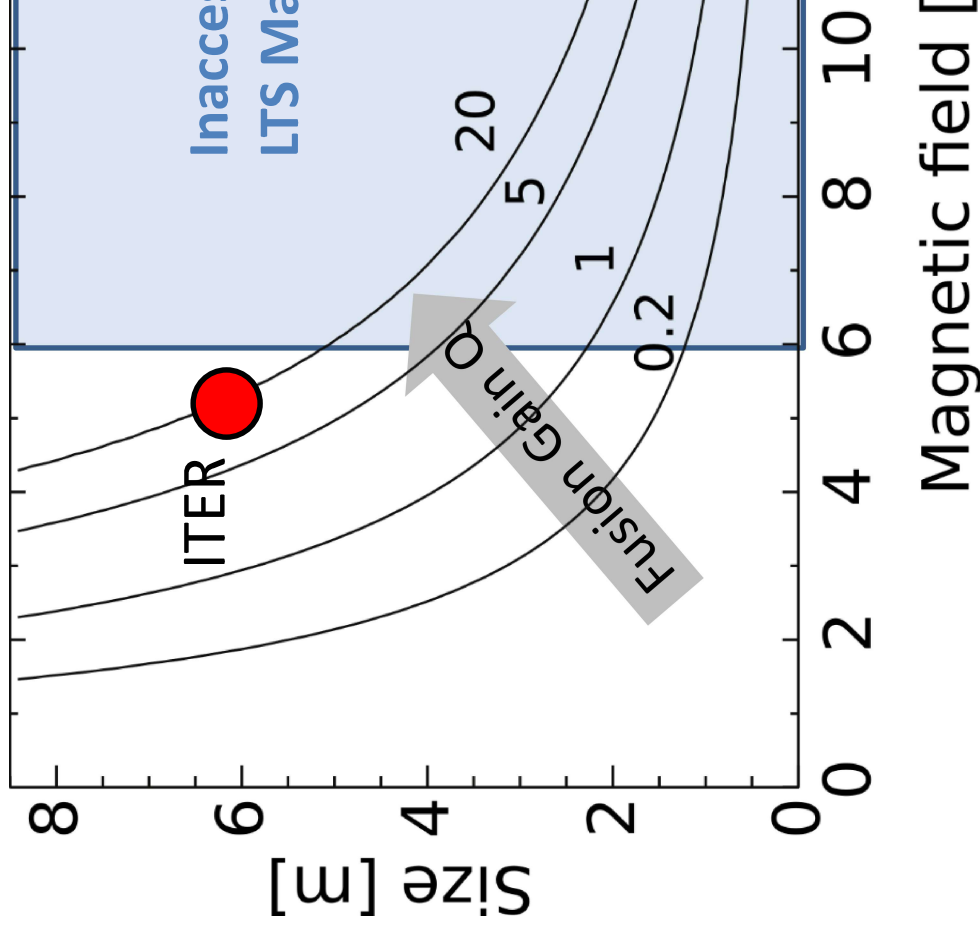
# At Higher Fields, Fusion Reactors Can Be

- The “size” of the plasma is properly measured in ion gyro-radii,  $R/\rho_i \propto BR$
- Higher power density
  - $R_{\text{Fusion}} \propto n^2 T^2 \propto p^2 \propto B^4$
  - Fusion gain:  $n\tau_E T \propto B^3$
- Using “Standard” physics assumptions we can map out fusion performance and see the quantitative trade-off between field and size
  - $H_{98} \sim 1$       –  $q \sim 3$
  - $A \sim 3$       –  $D/(D+T) \sim 0.5$
  - $\kappa \sim 1.8$



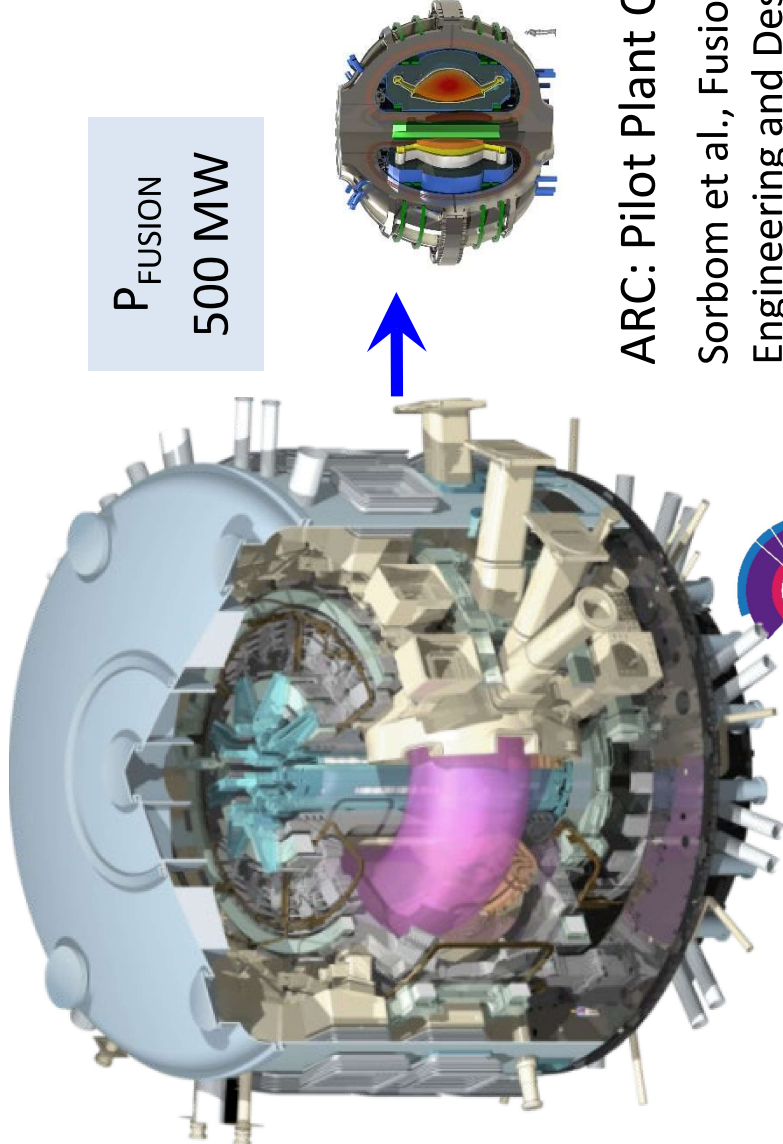
## But - With Conventional Superconductors, Machines Have To

- Practical fusion needs superconducting magnets
- “Best” LTS (Low Temperature Superconductors) is  $\text{Nb}_3\text{Sn}$
- Large-volume fusion magnets can't have much more than 5-6 T on axis
- That set the size for ITER
- Requires the combined resources of the entire industrialized world to build
- ~50 years from Reykjavik summit (1985) to DT operation (2035)

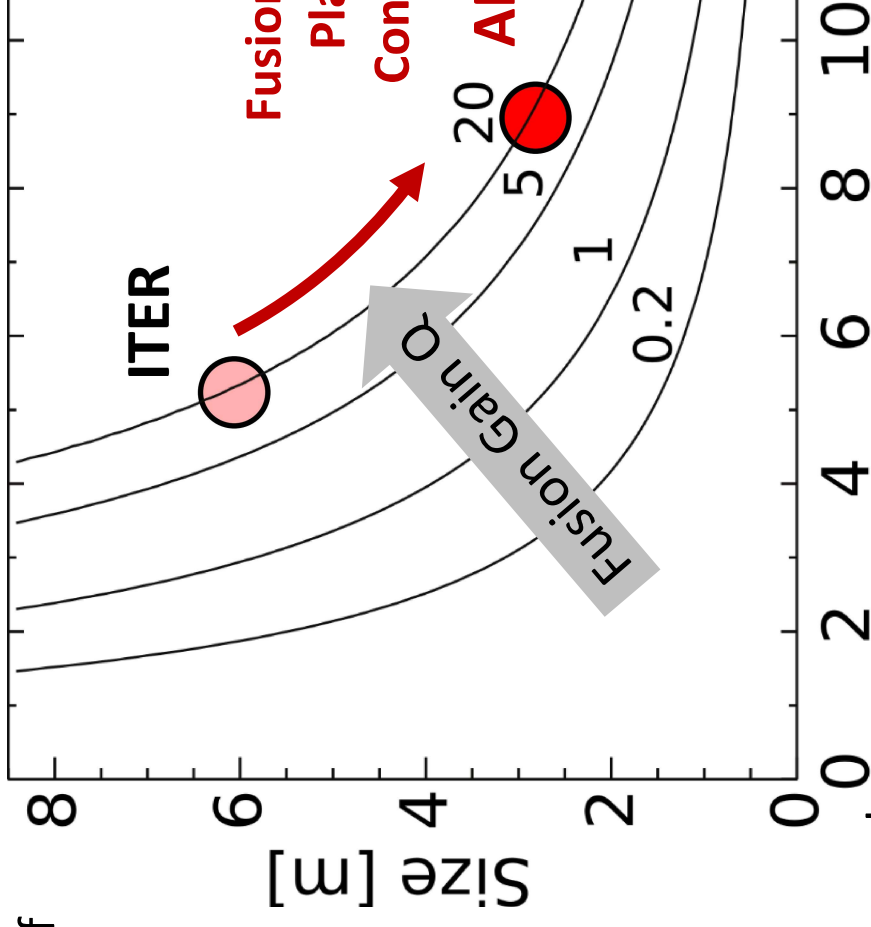


# The New HTS Magnet Technology Relaxes That Constraint At Higher Fields, We Can Go Smaller

- Doubling the field allows the size to drop in half
- The volume, weight decrease by a factor of 8
- Enables a new class of devices



$P_{\text{FUSION}}$   
500 MW



ARC: Pilot Plant Concept

Sorbom et al., Fusion  
Engineering and Design 2015

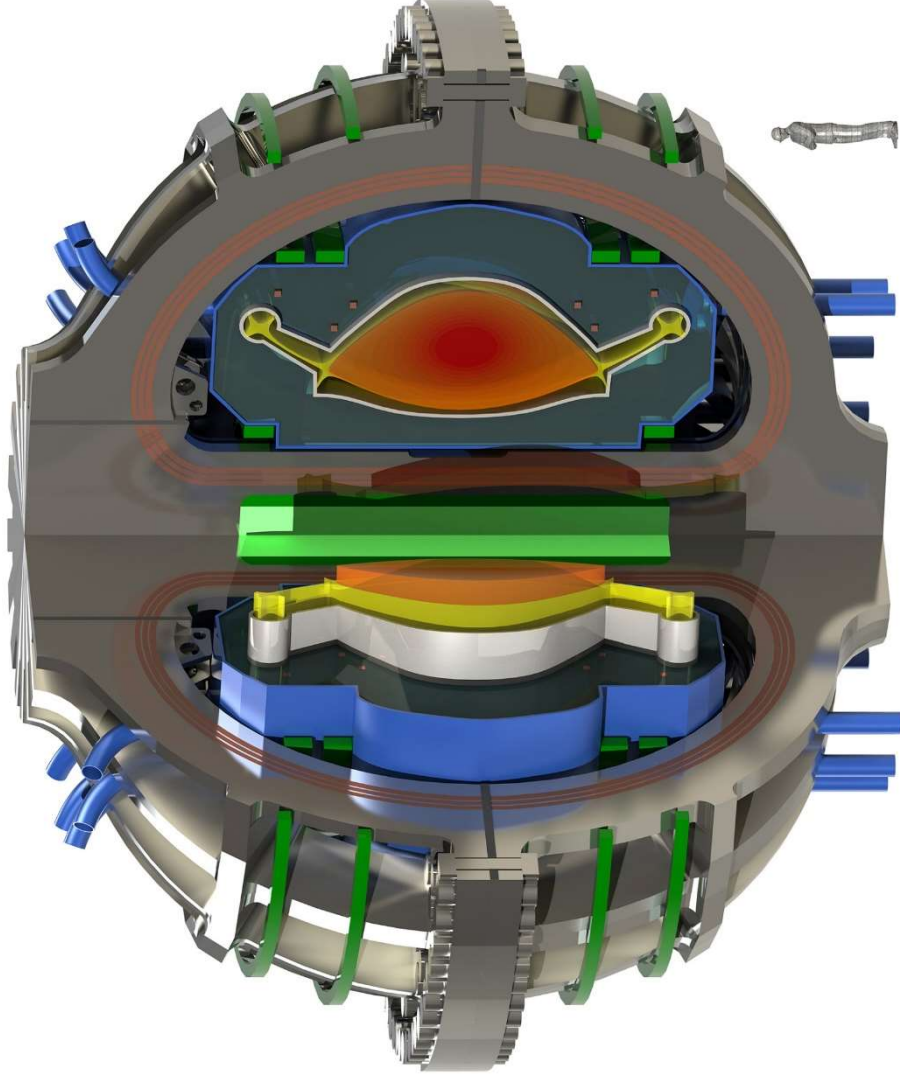


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# ARC: Concept for Class of Modular Fusion Pilot Plants



- Originated in a Graduate design class
- **Not** an engineering design (yet)
  - Though sufficient mechanical, hydraulic, electrical calculations were performed to demonstrate engineering plausibility for this class of compact modular fusion pilot plants

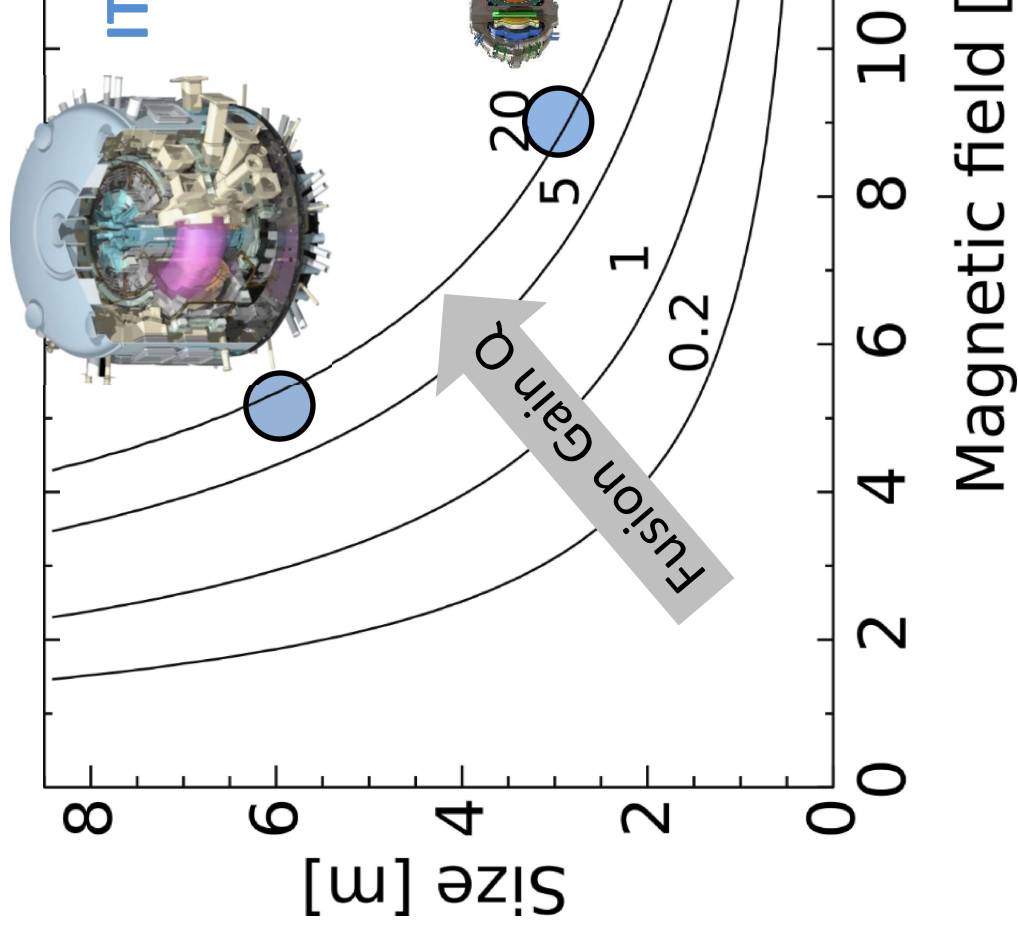
	ITER
R [m]	6.2
Magnet	LTS
B [T]	5.3
$P_{\text{fusion}}$ [MW]	500
$P_{\text{electric}}$ [MW]	0



# We're Not Ready To Build A Machine in the ARC Class

## SPARC: An Intermediate Step From Today's High-Field Experiments

- Burn DT to demonstrate  $Q_{\text{FUS}}$  robustly above 1 and with significant fusion power
- Same  $B_{\text{COIL}}$  as ARC – same technology
- Think of as a 12T, HTS version of AUG or DIII-D (currently operating expts.)
  - About 1/64 volume, weight, cost of ITER
  - Small enough to be built by a university/industry team



# SPARC Mission

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- **Make a plasma with (robustly) break-even fusion energy gain,  $Q > 2$** 
  - At long last, the Kitty Hawk moment for fusion
  - Our hypothesis – this would be a sufficient demonstration to put fusion firm on national energy plans and to attract investments for the next step
- **Demonstrate fusion-relevant HTS magnets at scale**
  - Integrated with high-performance tokamak operation
- **Provide the physics basis for high-field pathway**
  - Demonstrate high-field, burning fusion plasma scenarios





# That Is the Aspiration: What is the Plan?

## The SPARC Project

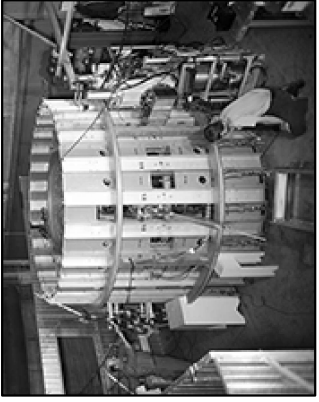
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- **Privately funded – official start March 2018**
  - Bolster fusion research and education at MIT while building a strong industrial (Commonwealth Fusion Systems - CFS) that aims to commercialize fusion energy
  - Combined team now ~ 240 people, capital raised so far \$215M
  - Go fast, accept risks, learn from mistakes
- **Phase I**
  - R&D & Demonstration of HTS fusion magnets **at scale**
  - Development of physics & engineering design for SPARC tokamak
- **Phase II**
  - Construction & Operation of SPARC
- **(Phase III): CFS Builds the world's first fusion pilot plant – an ARC-class device)**

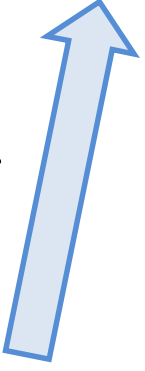


# Audacious? Yes, But Consider Some History...

ATC, 2T



July 1974:  
Tokamak physics  
mature *enough*  
to try DT

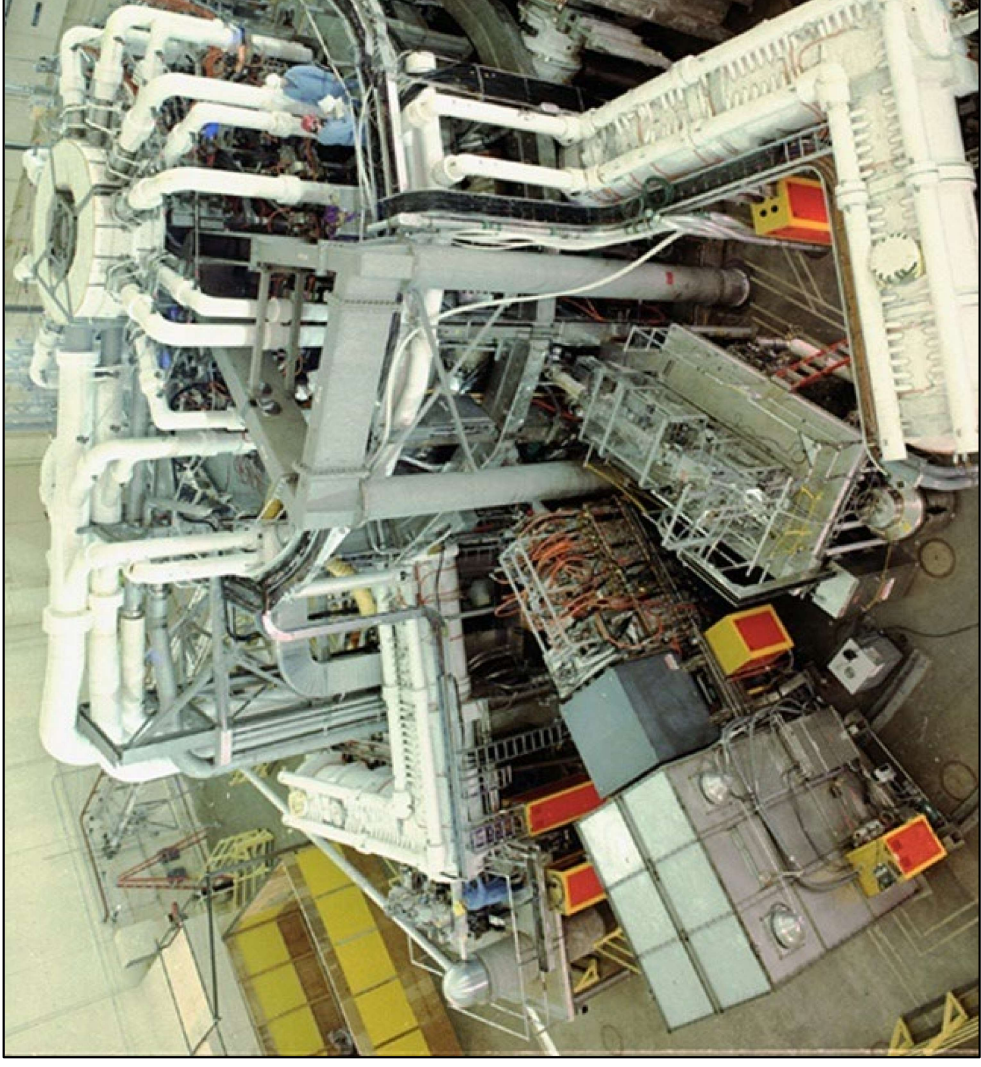


SPARC V2, 12T



2018:  
HTS mature  
*enough* to build a  
tokamak

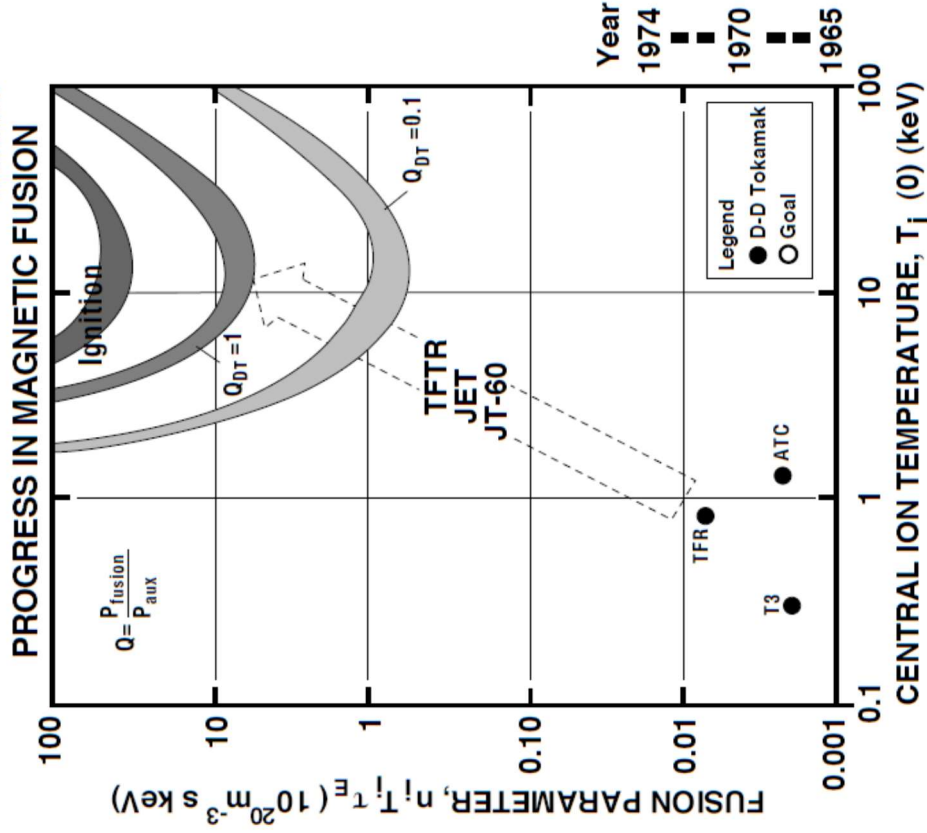
TFTR, 6T Construction approved 1976; 1<sup>st</sup> plasma



# A Huge Step From Where We Were in 1974

TFTR

PPPL#97GR085C



## Extrapolation factors from operating tokamaks

- Plasma current: 10 ✓
- Toroidal field: 2 ✓
- Magnetic energy:  $10^2$  ✓
- Pulse length:  $10^3$  ✓
- Auxiliary heating:  $10^2$  ✓
- Ion temperature: 10 ✓
- $nT\tau$ :  $10^3$  ✓
- Fuel: DT ✓
- Fusion power:  $10^8$  ✓
- Q:  $10^{8?}$  ✓

MGB / PPPL Colloquium / 141118



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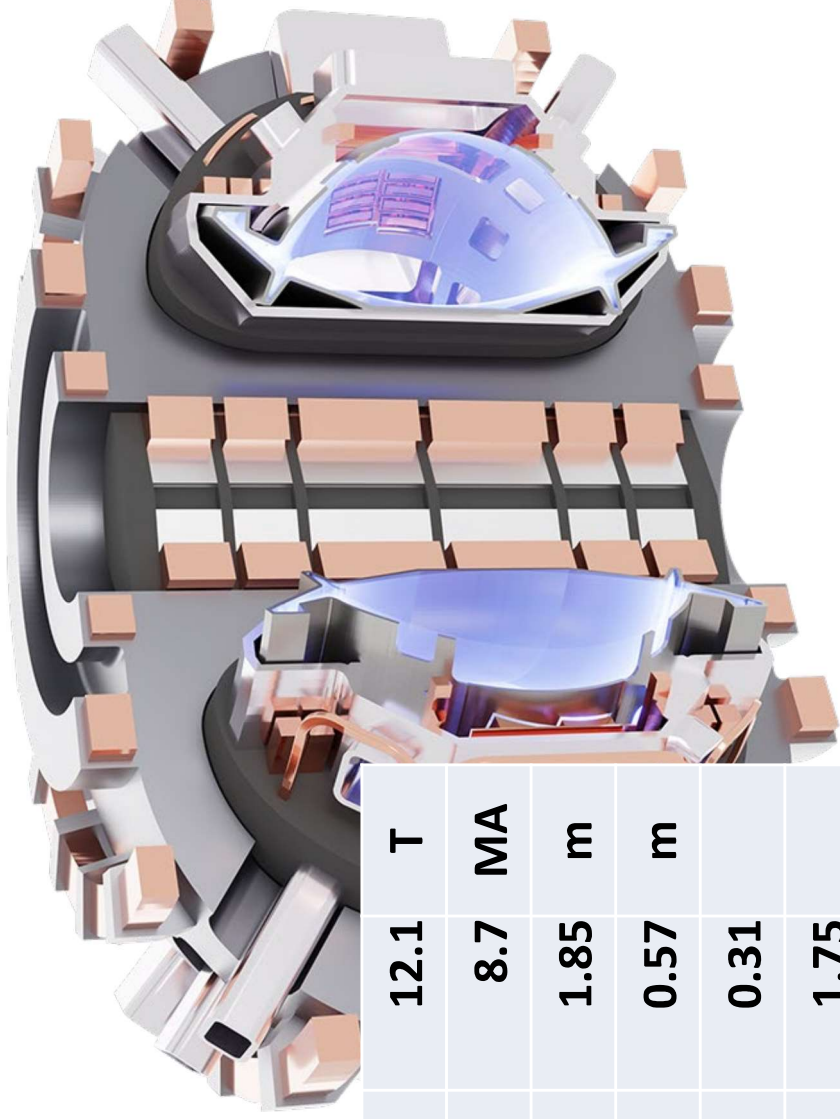
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Today, SPARC step compared decision to buy

SPARC extrapolation from operation

Plasma current	
Toroidal field	
Magnetic energy	
Pulse length	
Auxiliary heating	
Ion temperature	
Triple product	D-T fuel
Fusion power	
Q	

# SPARC V2: Starting Point for Detailed Engineering Design



$B_0$	12.1	T
$I_P$	8.7	MA
$R_0$	1.85	m
$a$	0.57	m
$\epsilon$	0.31	
$\kappa$ (area)	1.75	
$P_{fus}$	50-140	MW
$P_{ext}$	25	MW

## SPARC V0 technical requirements

- Burn **D-T** fuel
- Fusion Gain:  **$Q > 2$**
- **$P_{fusion} > 50\text{MW}$**  up to  $14\text{MW}$
- Pulsed with 10s flattop

## Desired schedule:

- R&D: 3yrs (mainly HTS)
- Construct: 4 yrs
- Operate: 5 yrs
- Decommission: 4 yrs

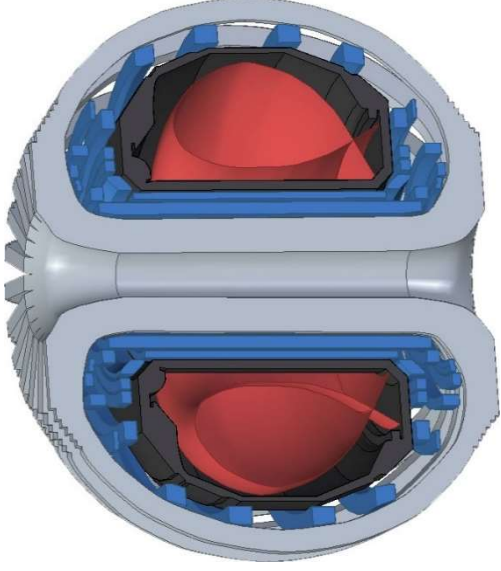
## Desired construction cost:

# SPARC – Similar In Size To Machines We’ve Built

DIII-D, General Atomics:

1.66m, 2.1T, <2.0MA

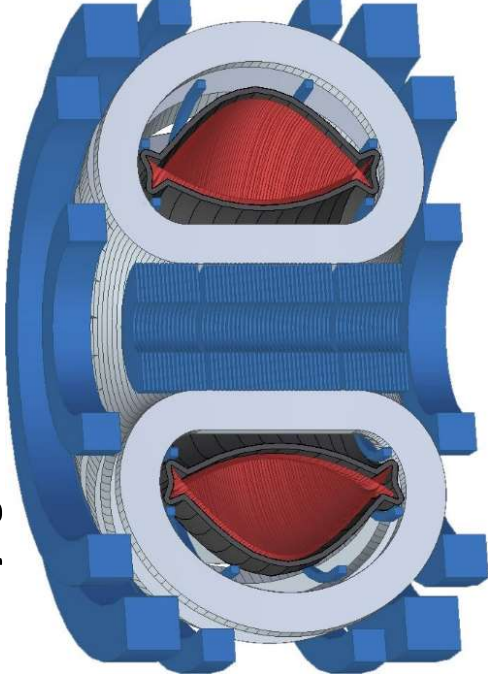
Water cooled Cu



SPARC V0, MIT:

1.85m, 12T, 8.7 MA

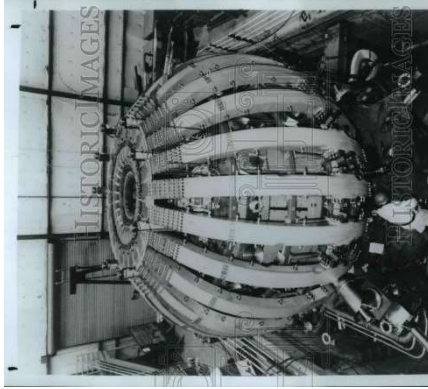
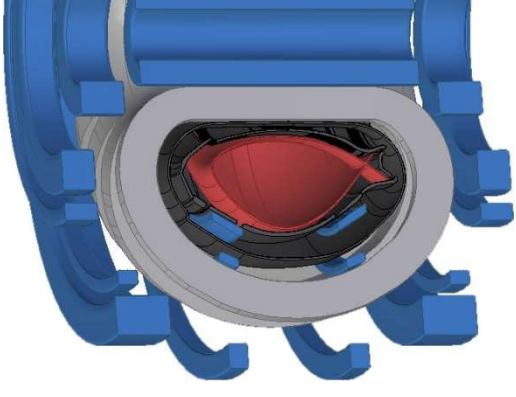
Cryogenic HTS



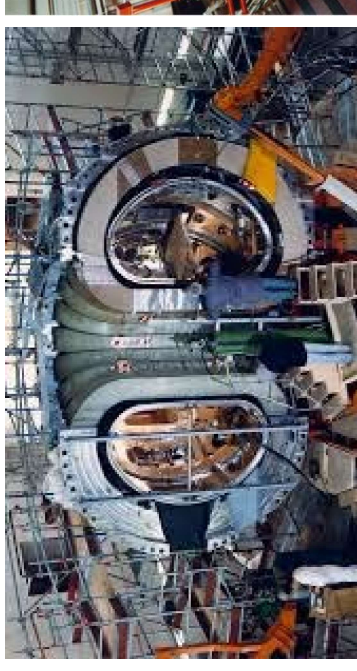
ASDEX-Upgrade, IPP G

1.65m, 3.1T, 1.6MA

Water cooled Cu



There are many others at this size: KSTAR, EAST, WEST, TEXTOR, PDX, PLT (and several bigger – TFTR, JET, JT60, JT60-SA, Tore-Supra)



# What Are The Principles Behind The SPARC Design Point

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- Use the ITER plasma physics basis
  - As far as possible, stay close to regimes and parameters where we have data
  - Be less of an extrapolation from existing experiments than ITER
- Use proven heating & control techniques
- Ensure robust operating windows
  - Stay away from known operational limits ( $\beta$ ,  $n/n_G$ )
  - Operate away regimes requiring complicated or expensive control systems
- Keep pulse short to avoid other engineering complications (current drive, heat removal) for the mission
  - But long enough for the plasma to reach steady state
- Keep nuclear envelope within fusion precedence (TFTR, JET) – no blanket



# Leverage As Much Mature Technology As Possible

## Mature knowledge utilized for SPARC

### Plasma Physics:

From 170+ tokamaks over the last 50+ years

### PF/CS design:

From FIRE, BPX, CIT

### Power supply design:

From C-Mod, Smaller than DIII-D, AUG

### Plasma heating design:

From TFTR, JET, FIRE, C-Mod

### Licensing:

From accelerators, TFTR, JET

### Heat exhaust:

From C-Mod

### Cryo systems:

From rocket industry

### Site size:

From TFTR/JET/FIRE

### Cell size:

From AUG

### Shielding:

From accelerators, TFTR, JET fission reactors

### Tritium handling, processing:

From TFTR, JET, LANL, Sandia, Bates

### Decommissioning:

TFTR, accelerators

## SPARC

### One Key Innovation

High Field Magnet

New HTS Technology



The SPARC HTS  
(12 T on axis;

# SPARC: How Do We Estimate Fusion Performance?

---

- **Hierarchy of models – No closed-form solutions for turbulent transport**
- **Empirical Scaling: 0D – Transport is characterized by  $\tau_E = \text{energy confinement time}$** 
  - For ITER design, a large data set was assembled
  - Regress  $\tau_E$  against “engineering” parameters (plasma size, input power, magnetic field,
- **Dimensionless identity:**
  - Plasma physics equations can be cast in dimensionless form
  - With identical dimensionless parameters, plasma physics should be identical
- **Time-dependent simulation – quasi-linear transport model**
  - Quasilinear = calculate linear growth rates at all wavenumbers and normalize transport saturation “rule” calibrated to nonlinear GK simulations
- **Nonlinear gyro-kinetic simulations**
  - GK = Boltzmann equation averaged over gyro-motion (equation of motion in 5D phase space)



**SPARC**

SPARC

LLNL Seminar: 03/25/21



# 0D Analysis From Empirical Scaling Laws

- **Use ITER Performance Rules**
  - Energy Confinement:  $H_{98} = 1$
  - Profile peaking factors
  - Fuel mix
  - Fuel dilution

## • **Operating Space Defined by**

- $Q_{\text{FUSION}} > 2$
- $P_{\text{LOSS}} > P_{\text{L-H}}$  (Threshold)
- $P_{\text{HEATING}} < 30 \text{ MW}$
- $P_{\text{FUSION}} < 100 \text{ MW}$

## Calculation Proceeds in the $\langle n \rangle$ , $\langle T \rangle$

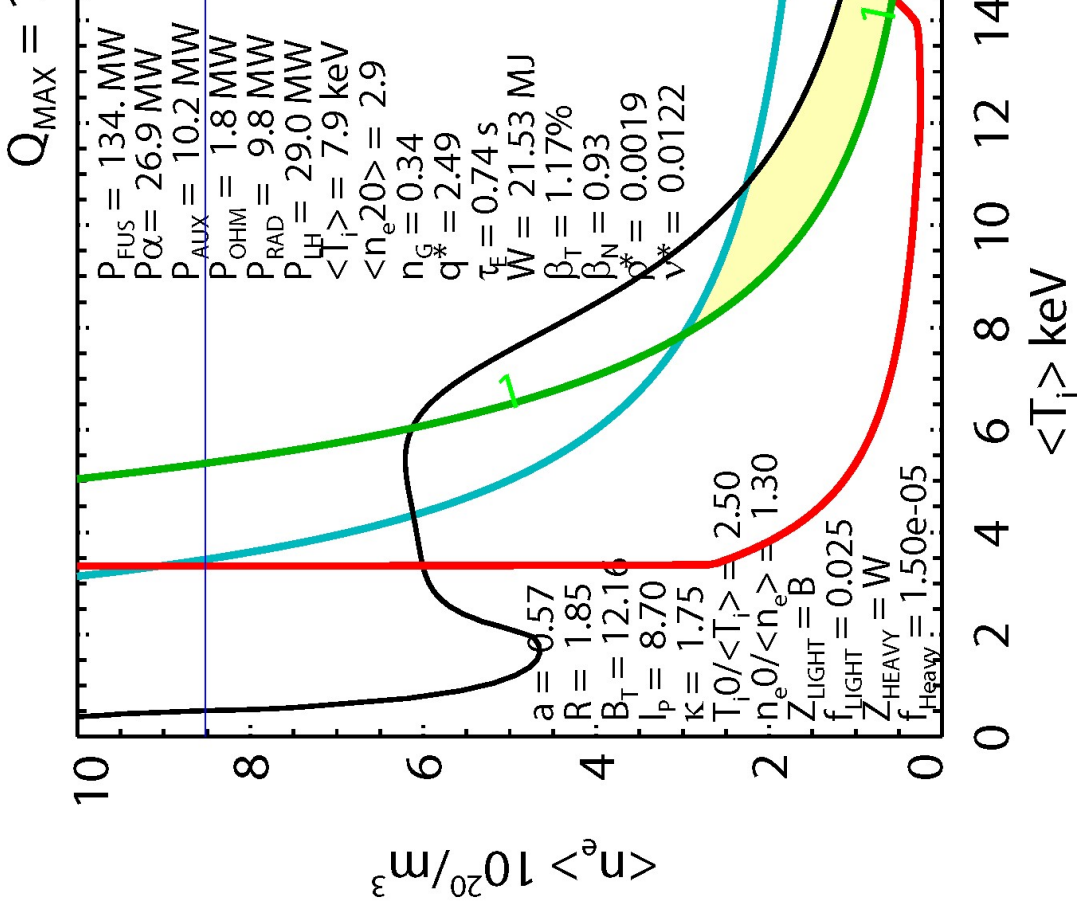
1. Compute fusion power using  $n$ ,  $T$ , fuel dilution, profile peaking factors
2. Compute radiated power (Bremms, L radiation, synchrotron) using  $Z_{\text{EFF}}$ , plasma  $B_T$ , profile factors
3. Compute total input power required confinement law (from empirical scaling)
4. Compute aux power required from power balance  $P_{\text{AUX}} + P_{\alpha} + P_{\text{OH}} = P_{\text{Loss}} + P_{\text{RA}}$
5. Compute operational limits ( $\beta$ ,  $n/n_G$ ,



SPARC

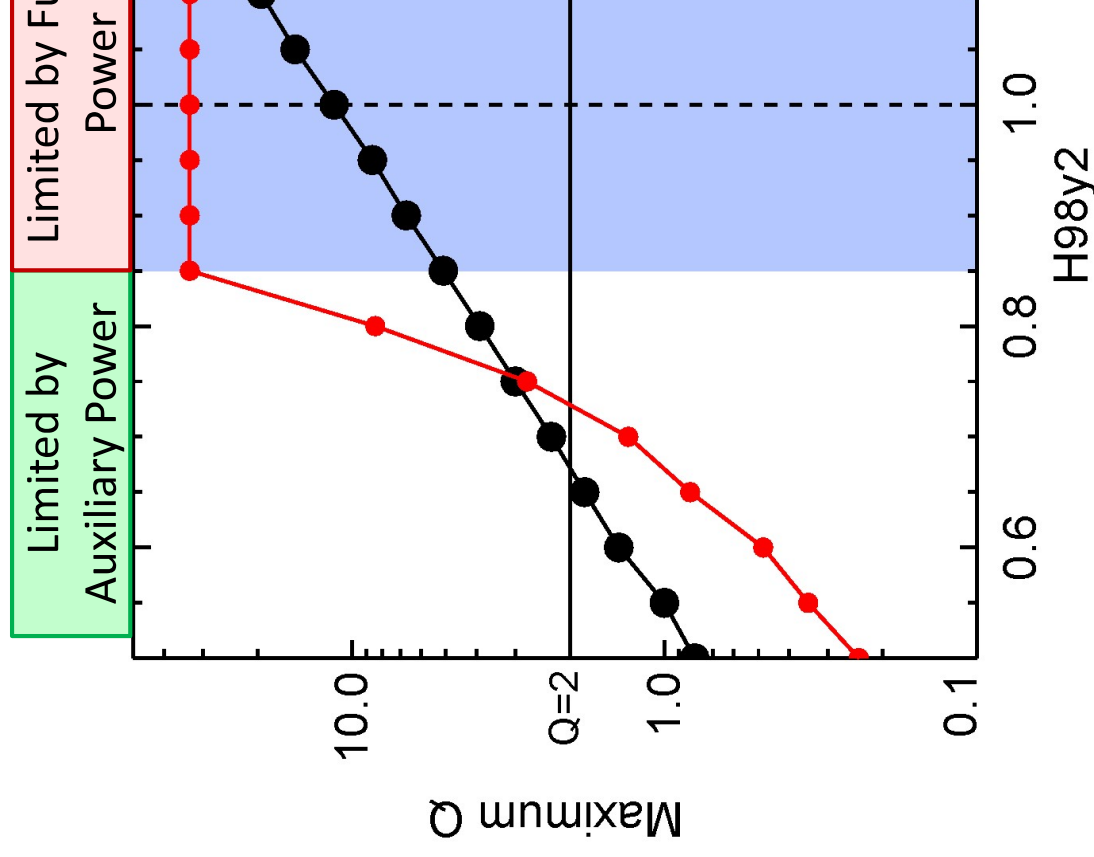
# SPARC: Nominal Operating Space; $Q_{FUS}$ up to 11

- Use ITER Performance Rules
- Energy Confinement:  $H_{98} = 1$
- Profile peaking factors
- Fuel mix
- Fuel dilution
- Operating Space Defined by
  - $Q_{FUSION} > 2$
  - $P_{LOSS} > P_{L-H}$  (Threshold)
  - $P_{HEATING} < 25$  MW
  - $P_{FUSION} < 140$  MW

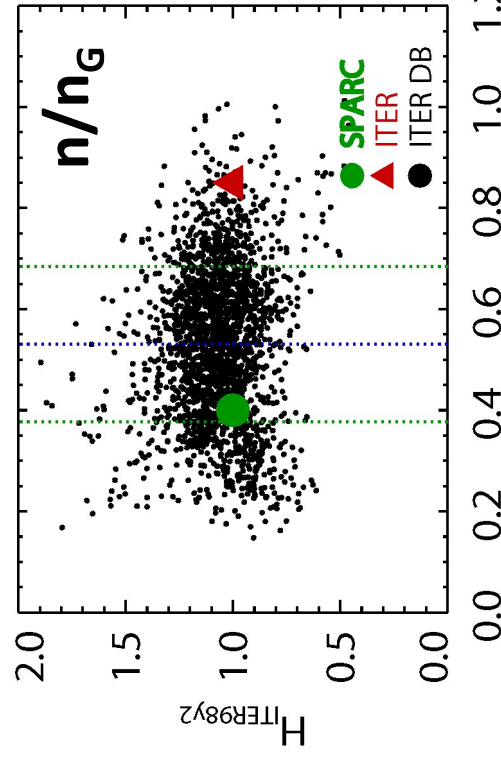
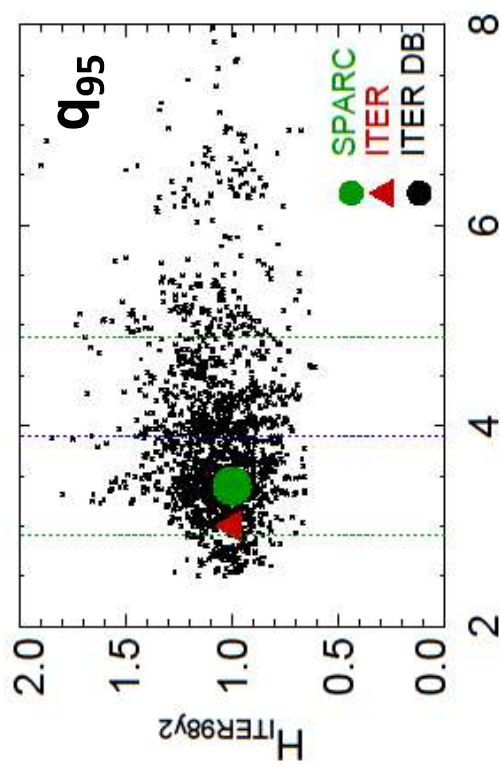
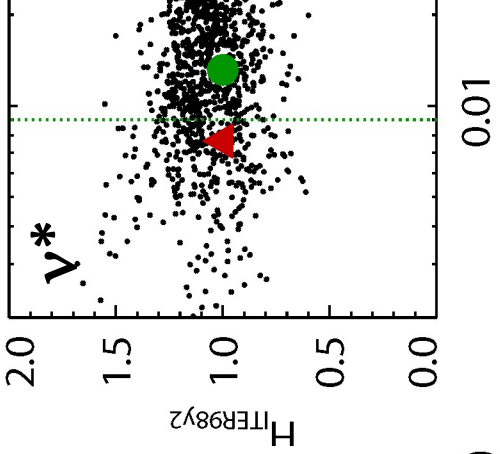
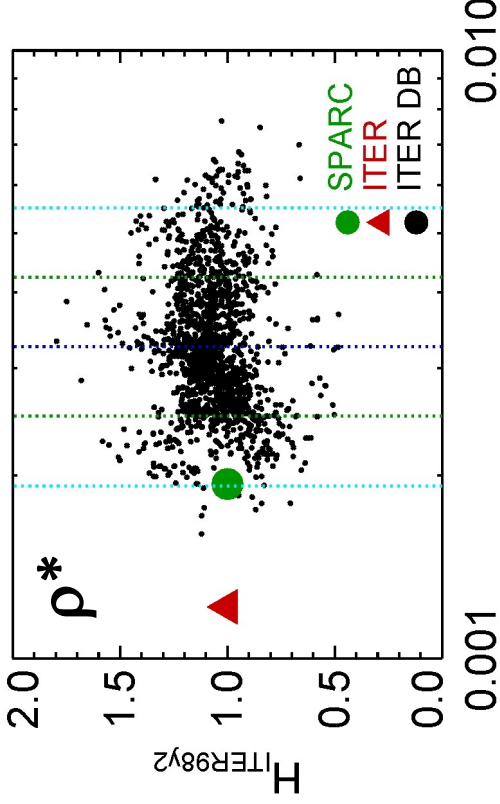
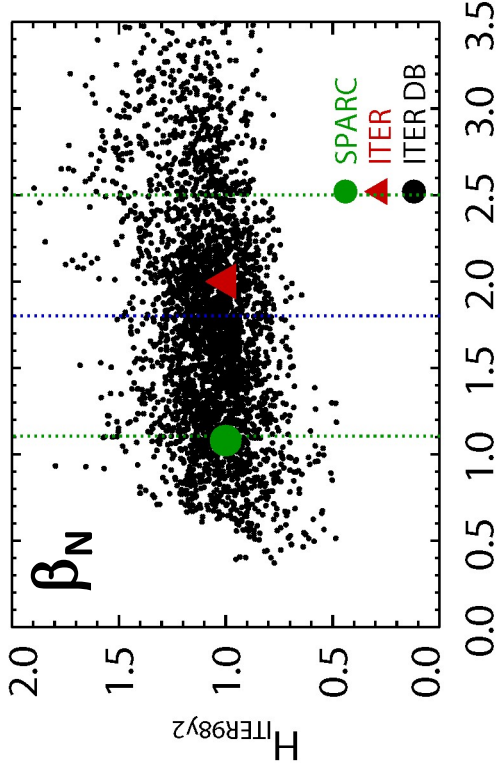


# As Planned, SPARC Has Performance Margin

- Limited by auxiliary power at low confinement
- Engineering limits on fusion power output at high confinement
- Exceeds  $Q=2$  at  $H \approx 0.7$
- Provides margin against confinement uncertainty or
- Reduced pedestal height
  - i.e. margin against peeling-ballooning (ELM suppression)



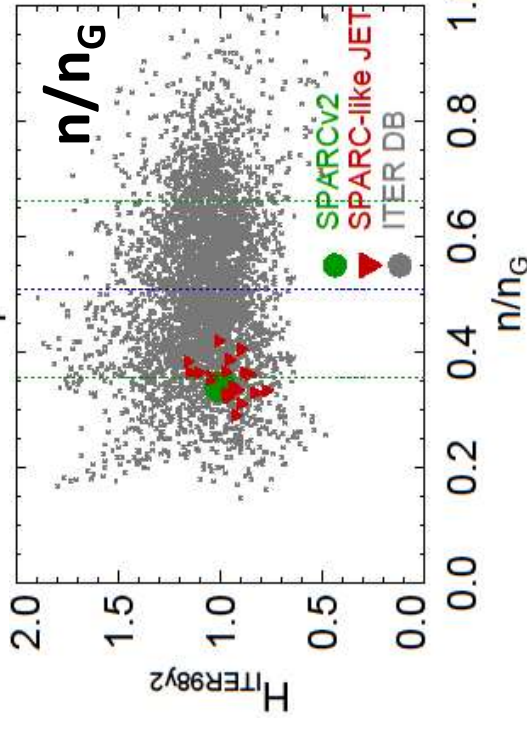
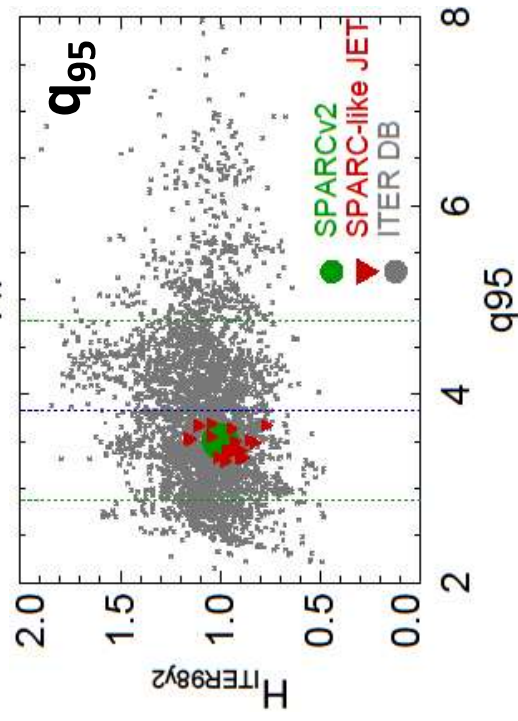
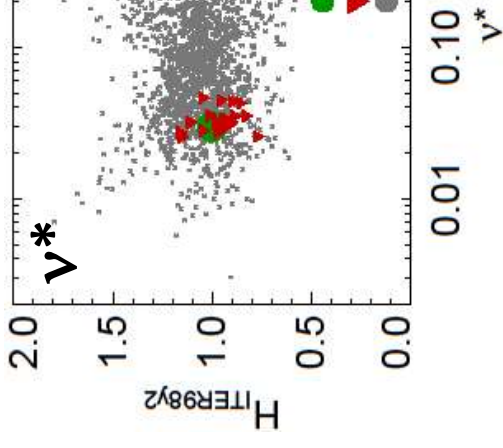
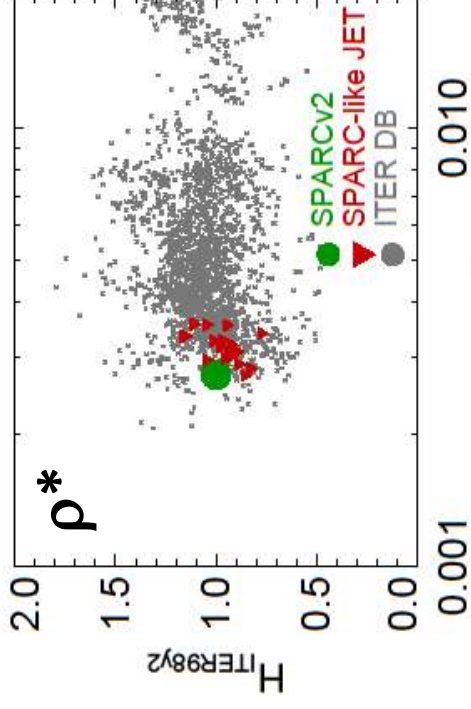
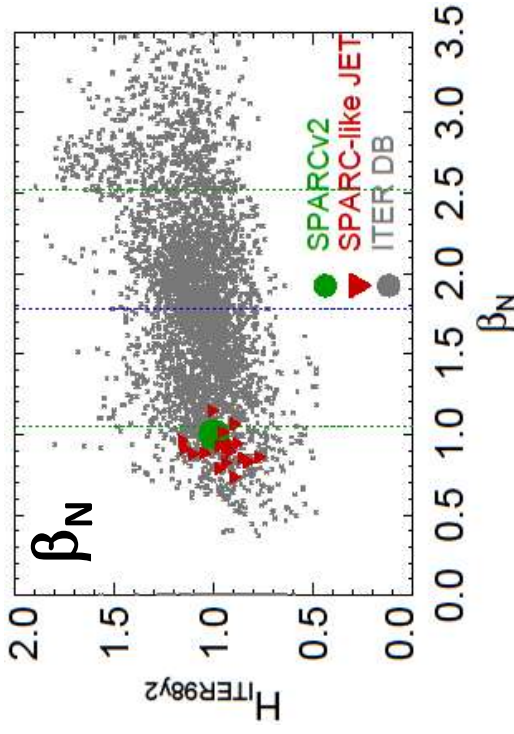
# In Dimensionless Variables, the SPARC Operating Point Is Closer to the Mean of the H-mode Confinement Database than



- ITERDB data from  $H_{98}$  scaling like geometrical
- Exception: some contain the kinetic energy required to calculate



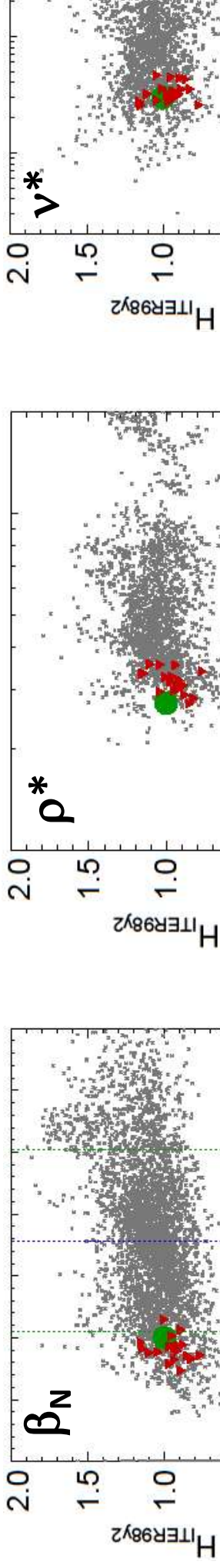
# We Can Find Discharges That Are Very Close To Matching All SPARC Dimer Plasma Parameters ( $\beta_N$ , $v^*$ , $\rho^*$ , $q_{95}$ , $n_G$ , $\varepsilon$ , $\kappa$ , $\delta_I$ ) Simultaneously



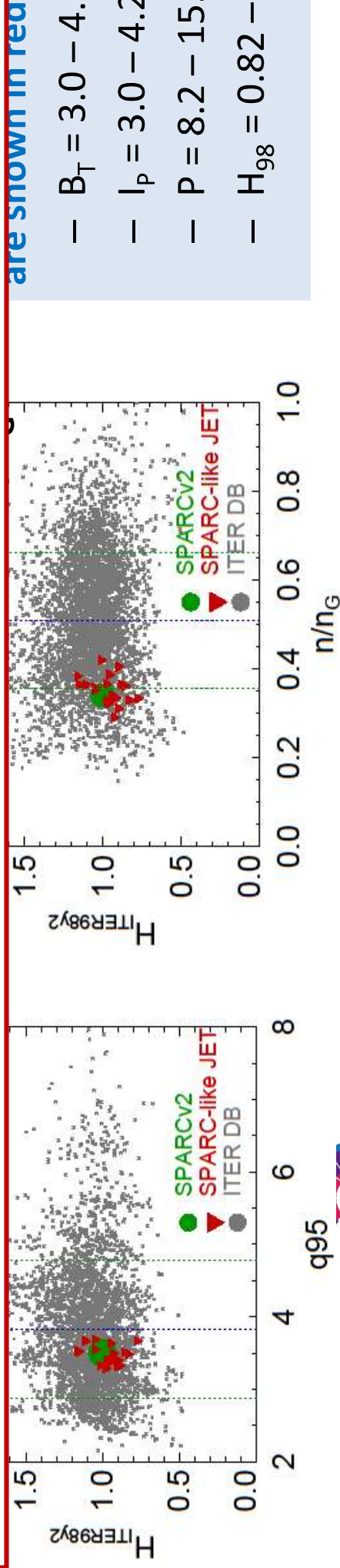
The same 20 (JET) are shown in red

- $B_T = 3.0 - 4.0$
- $I_p = 3.0 - 4.2$
- $P = 8.2 - 15$
- $H_{98} = 0.82 -$

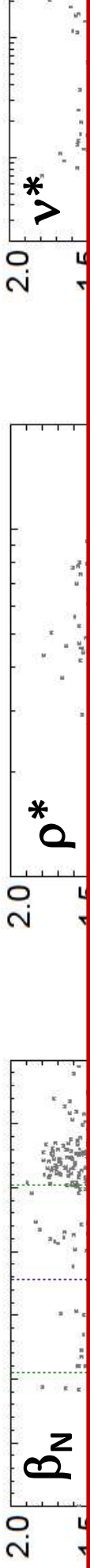
# We Can Find Discharges That Are Very Close To Matching All SPARC Dimer Plasma Parameters ( $\beta_N$ , $v^*$ , $\rho^*$ , $q_{95}$ , $n_G$ , $\epsilon$ , $\kappa$ , $\delta_I$ ) Simultaneously



Thus: Much of the Core Plasma Physics Has Been Already Covered



We Can Find Discharges That Are **Very Close To Matching All SPARC Dimer Plasma Parameters** ( $\beta_N, v^*, \rho^*, q_{95}, n_G, \epsilon, \kappa, \delta_I$ ) Simultaneously

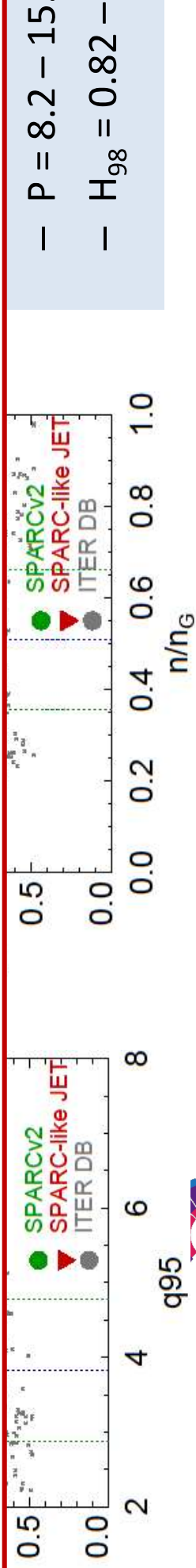


**Why didn't those JET discharges generate 100 MW of fusion power?**

Fusion is nuclear physics – Doesn't scale with dimensionless plasma parameters

It cares about  $T/E_{Nuclear}$

- In fact, we're eagerly looking forward to experiments in the regime where nuclear physics and nuclear physics are coupled – **this will be new**



- $P = 8.2 - 15$  MW
- $H_{98} = 0.82 - 1.0$



## Ensuring Performance : Time-dependent Plasma Simulation

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- Compared to empirical scaling which is **ALL DATA** and minimal physics
- These simulations are **ALL PHYSICS** (reduced models where necessary) **minimal data**
  - 2D MHD equilibrium, (1D radial profiles = 1.5D)
  - Plasma Profiles - Quasi-linear transport to calculate
  - Heating - RF wave propagation & damping
  - Heating – Fusion reaction rate and alpha power deposition
  - + lots more...

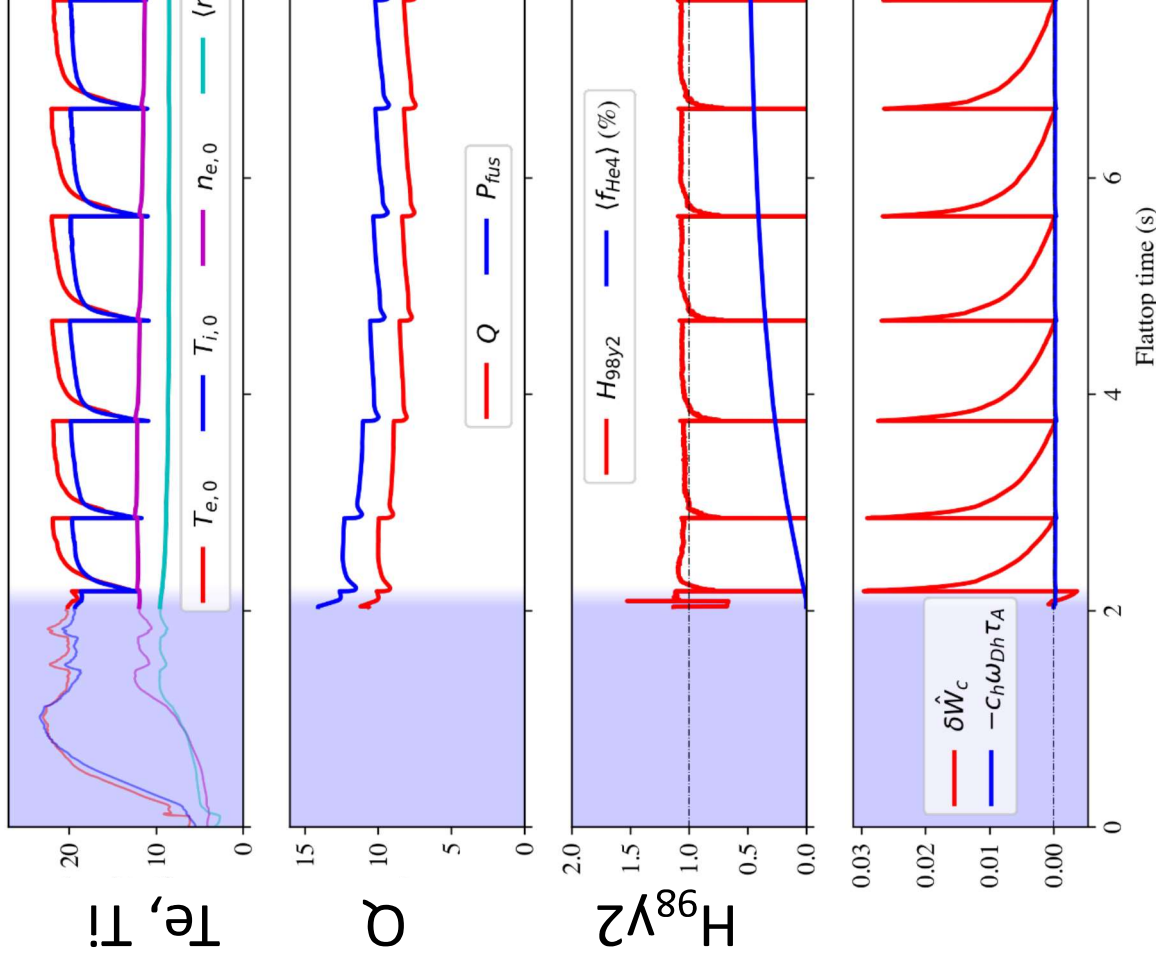




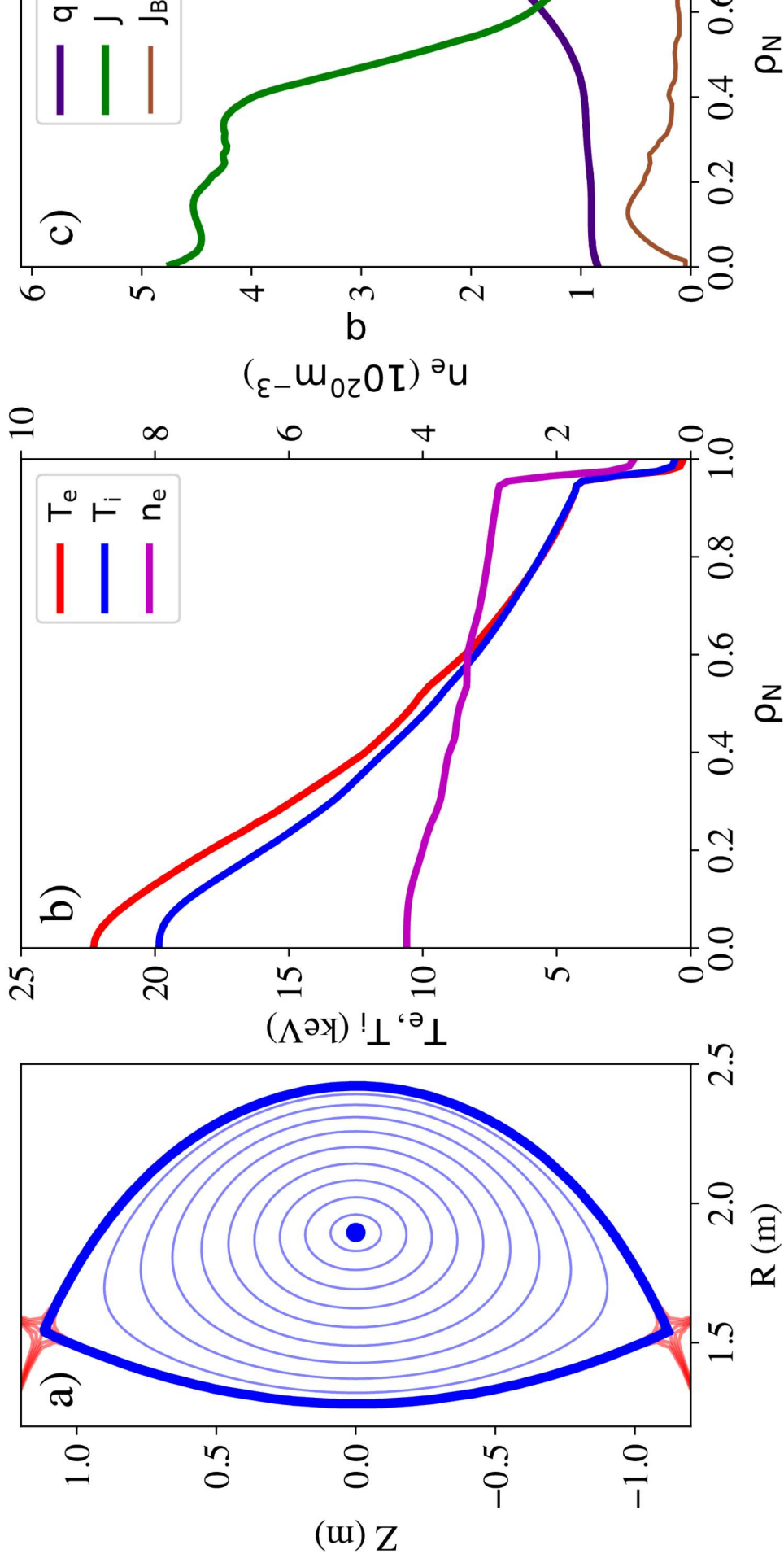
# Remarkable Agreement Between 0D and 1.5 D Time-dependent

- For this calculation, the energy confinement time, H factor, temperatures, fusion power, Q are all outputs
- Methods are entirely independent
- However, results are remarkably close

	TRANSP	Empirical
$H_{98}$	1.0	1.0
$\langle Ti \rangle$	7.8 keV	7.9 keV
$Ti(0)/\langle Ti \rangle$	2.5	2.5
$P_{RAD}$	13.2 MW	10.4 MW
$P_{FUS}$	105 MW	140 MW
Q	9	11

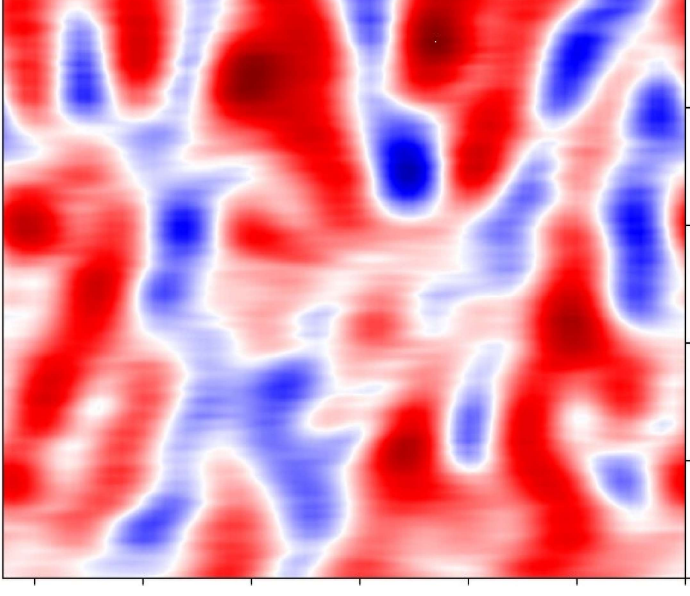


# 1.5 D Predictions of Plasma Profiles For SPARC



## Next Steps - Nonlinear Simulation of Gyrokinetic Turbulence

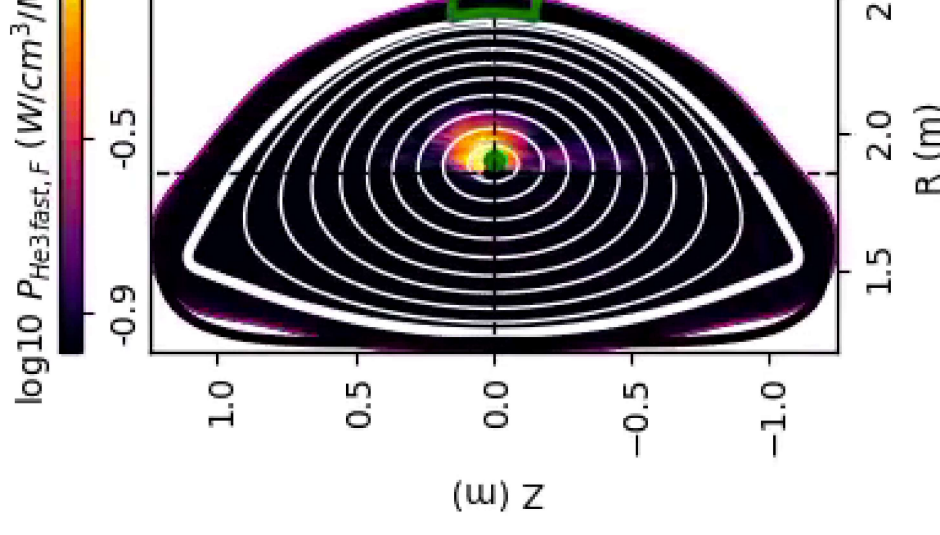
- Quasi-linear model has been benchmarked against experiments and gyrokinetic codes
- We've sampled the profile at a few points and looked at the expected turbulence
  - No surprises – SPARC should be in turbulence regimes similar to other devices
- Still, we want to take the next step
- We're in the process of launching a fully nonlinear gk prediction of profiles – a “heroic” simulation
  - Constrain gradients at each radii with nonlinear GK



# Getting Power In: Plasma Heating - 25 MW ICRF

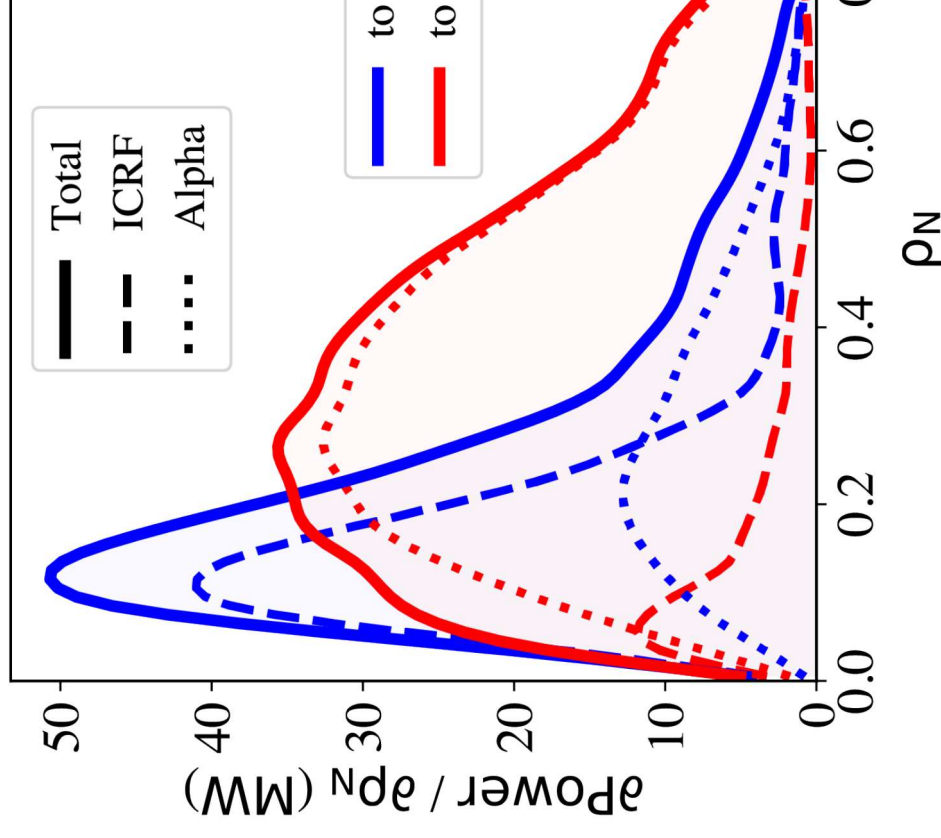
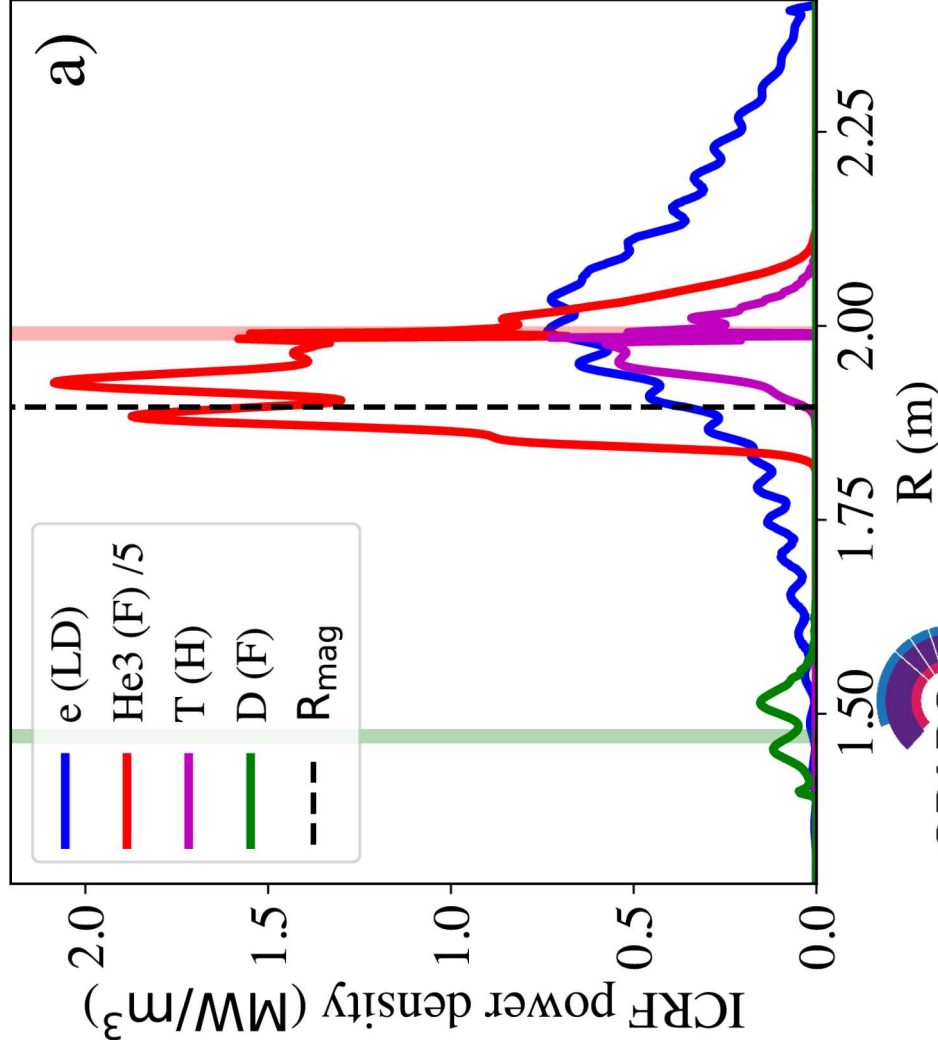
- **Simplicity: Utilize a single type of heating**
  - Proven to work on C-Mod (at required plasma density) and on DT devices (TFTR & JET)
- **Heating scenarios at 120 MHz look promising**
  - Options for DD (H minority at 8T)
  - DT (He<sup>3</sup> minority at 12T) operation (+ perhaps 2<sup>nd</sup> harmonic T)
  - Good single pass absorption predicted
- **Optimum antenna spectrum is a balance between coupling and absorption**

ICRF scoping calculations

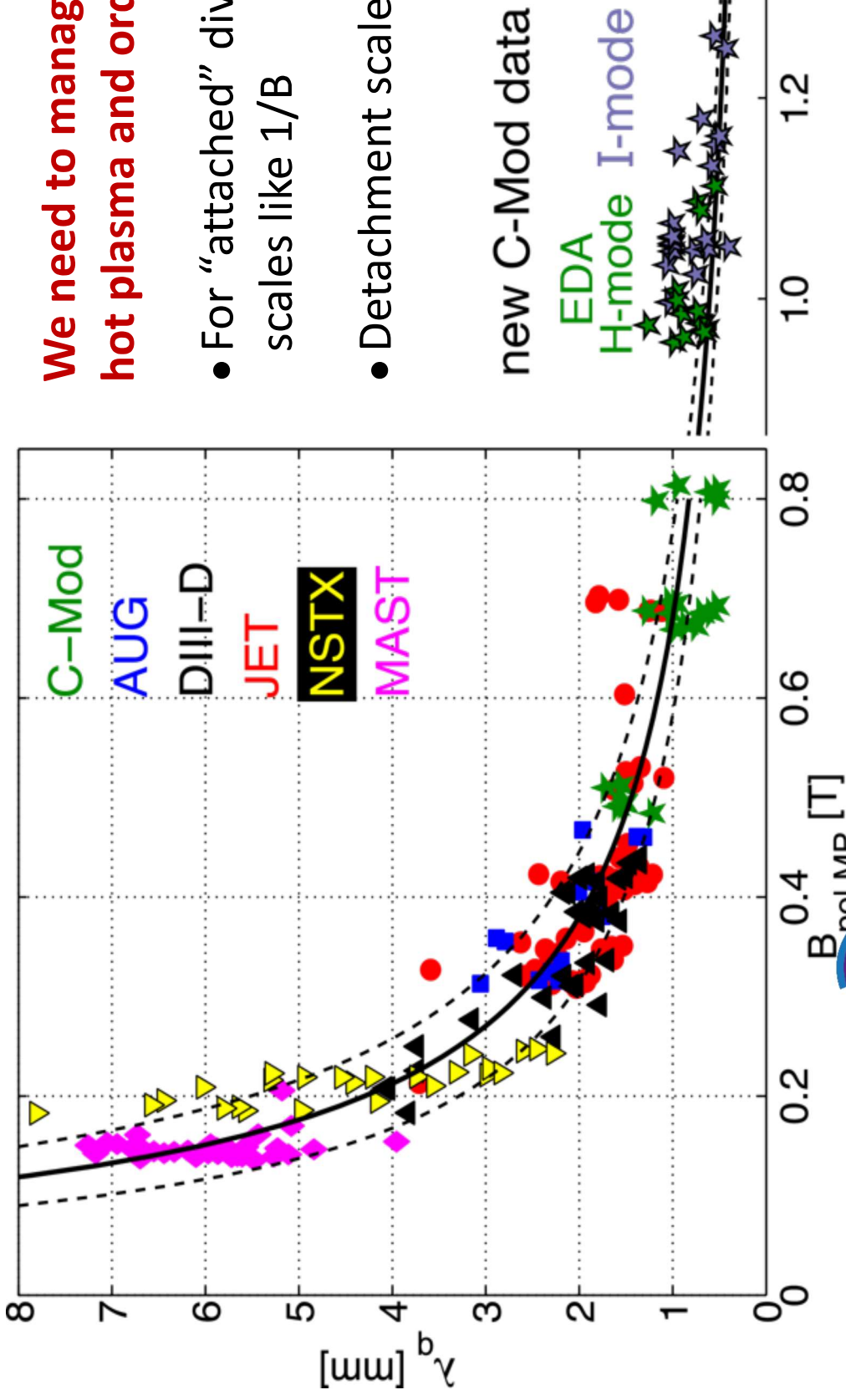


# ICRF Heating Is Well Localized In Plasma Core

## Most ICRF Power into Ions, Alpha power to electrons



# Getting Power Out: Divertor Loading –A Challenge At High Perf



**We need to manage the interface between hot plasma and ordinary matter**

- For “attached” divertor, the heat flux scales like  $1/B$
- Detachment scales like  $PB/Rn_{SEP}^2$

new C-Mod data

EDA  
H-mode I-mode

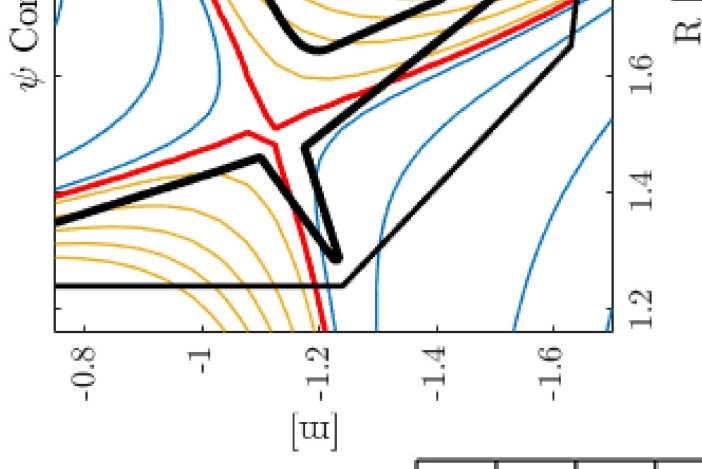
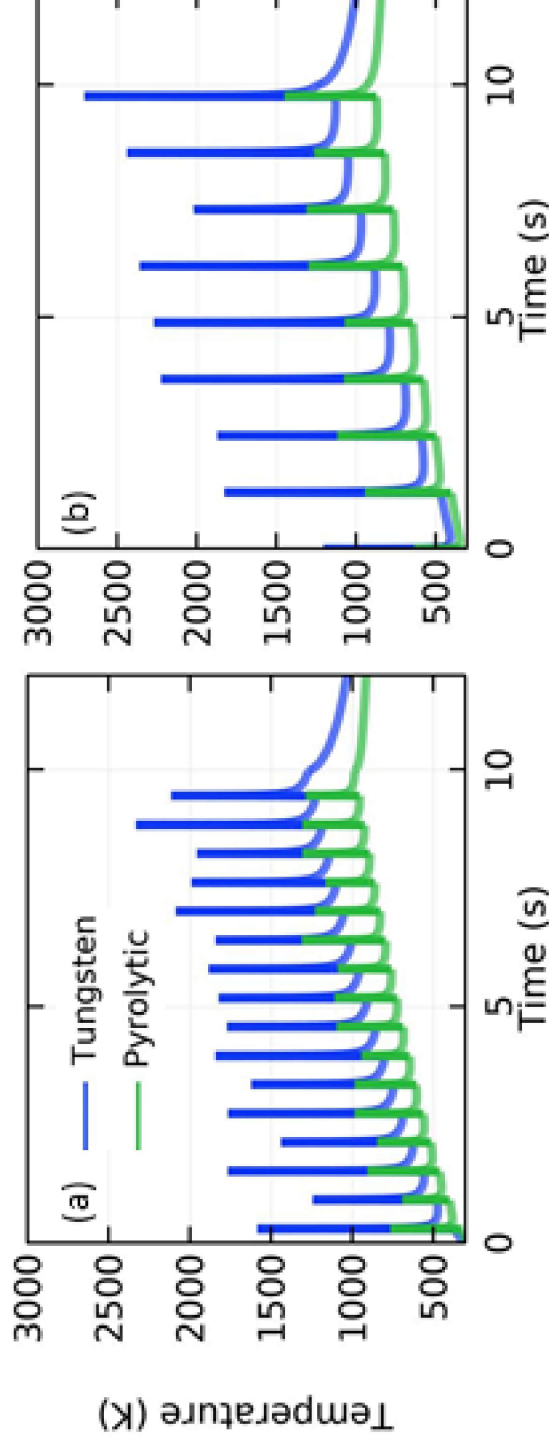
(Though recent studies suggest that this is beneficial, I don't count this)



Eich, et al., JNM 2011 Brunner, 2017

# Boundary power exhaust challenging but not a show-stopper for SPARC Simulations show that heat flux can be handled with aggressive strike point

- With standard (C-Mod, ITER) vertical target divertor, 10 sec pulse
  - At typical and acceptable radiation fraction (50%), target materials survive full power pulse. (assumes fully attached, single null)
  - Radiation fractions of > 90% have been demonstrated on C-Mod, AUG, JET while maintaining good core confinement



# Disruptions: A Challenge For All High-Performance Tokam Compact, High-Field Has Some Advantages

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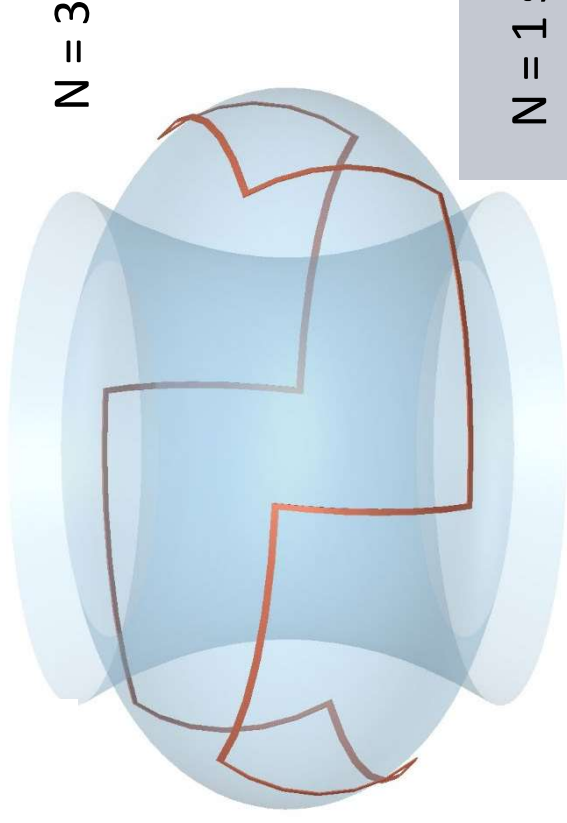
- **Operating away from limits**
  - ↑ Staying further away from limits (especially pressure, density) should reduce disruptive
- **Disruption forces are not obviously more difficult for compact, high-field**
  - IxB and stresses about the same
  - ↑ distances can be smaller for mitigation actuators (gas, pellets)
  - ↓ But, quench times faster for compact devices
- **Runaway electron growth rate reduced, damping enhanced**
  - ↑ Runaways more strongly damped by collisions and synchrotron radiation
- ↑ Growth (Coulomb Avalanche) exponential with plasma current ( $I_{runaway}/I_{seed}$ )  $\propto e^2$



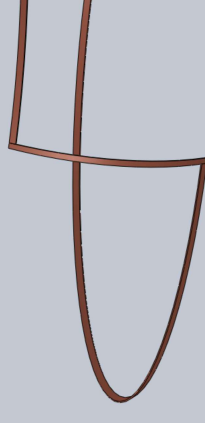


# To Mitigate Disruption Runaways, We Will Try Out A Novel 3

- From an original idea by A. Boozer
- Current driven inductively by  $dI_p/dt$  of disruption
- Non-axisymmetric coil creates 3D fields which break up flux surface, leading to rapid loss of “seed” electrons
- Analysis of plasma response by V. Izzo – with NIMROD – extended MHD code
- **Looks very promising**
- **Offers possibility of passive safety wrt runaways!**

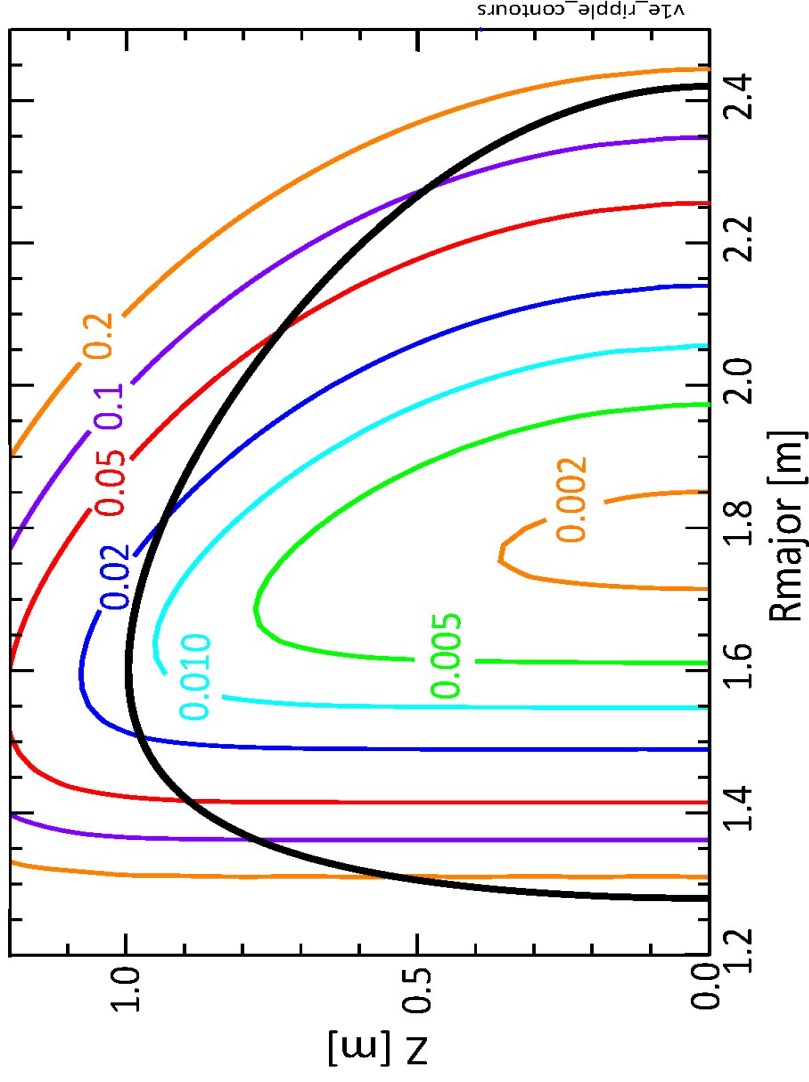


N = 1 seems to work best



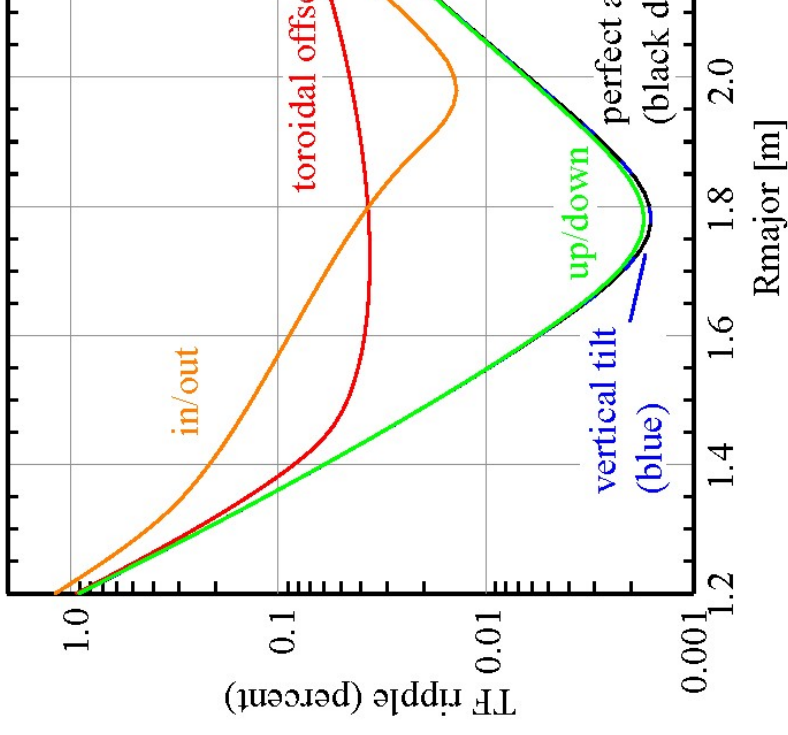
# Loss of Fast Alphas from Ripple Transport Should Be Minimized

Ripple contours (%) 18 coils perfect alignment



- Ripple loss 0.25 %
- Neoclassical loss 2.8%

Ripple Profile With Significant Toroidal Offset



- Ripple loss 1 % (all from toroidal offset)



SPARC

# How About Transport of Fast Alphas by MHD?

- Perhaps the most important new physics that the SPARC experiment will address
- The fully self-consistent, nonlinear interaction of fast particle distributions and MHD in a plasma is a cutting edge research problem – need a burning plasma experiment to validate
- We've done some linear analysis ([Tolman, et al., Nucl. Fusion 59, 046020, 2019.](#))
  - $n_{\alpha}/n_e \propto T^{3.5}$
  - Drive to damping ratio for Alfvénic modes  $\propto \beta_{\alpha}/\beta_e \propto T^{2.5}$
  - Bottom line – with similar temperatures, alpha physics in SPARC will be similar to what expected on ITER - good for baseline H-mode operation
  - Higher field will push the spectrum of AE modes to higher  $n$ ,  $m$
- Diagnostics are planned to look at fusion rate, confined alpha distributions and fast MHD
  - Neutron detectors and neutron spectrometers look like promising tools



## There's Lots More Physics – See JPP Special Issue on SPARC Physics

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**Overview of the SPARC tokamak, A.J. Creely et al,**

J. Plasma Phys. 86 (5), 865860502 (2020). <http://doi.org/10.1017/S0022377820001257>

**Predictions of core plasma performance for the SPARC tokamak, P. Rodriguez-Fernan**

J. Plasma Phys. 86 (5), 865860503 (2020). <https://doi.org/10.1017/S0022377820001077>

**Projections of H-mode access and edge pedestal in the SPARC tokamak, J.W. Hughes**

J. Plasma Phys. 86 (5), 865860504 (2020). <https://doi.org/10.1017/S0022377820001300>

**Divertor heat flux challenge and mitigation in SPARC, A.Q. Kuang et al,**

J. Plasma Phys. 86 (5), 865860505 (2020). <https://doi.org/10.1017/S0022377820001111>

**Physics basis for the ICRF system of the SPARC tokamak, Y. Lin et al,**

J. Plasma Phys. 86 (5), 865860506 (2020). <https://doi.org/10.1017/S0022377820001266>

**MHD stability and disruptions in the SPARC tokamak, R. Sweeney et al,**

J. Plasma Phys. 86 (5), 865860507 (2020). <https://doi.org/10.1017/S0022377820001122>

**Fast-ion physics in SPARC, S.D. Scott et al,**

J. Plasma Phys. 86 (5), 865860508 (2020). <https://doi.org/10.1017/S0022377820001088>

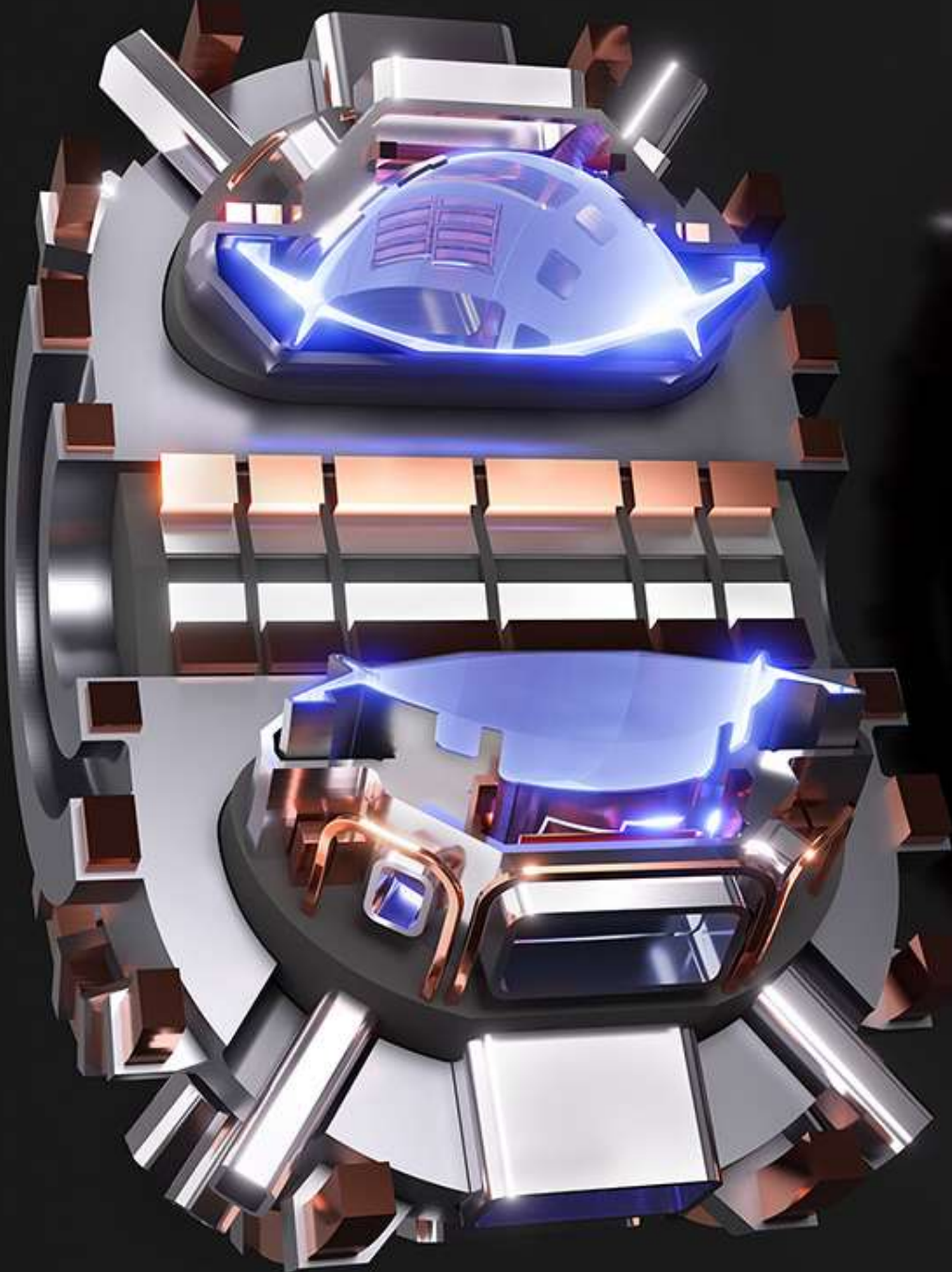


## What's Next?

# Device Optimization, Physics Validation, Engineering Review

- Detailed engineering underway
- Deeper dive into physics validation
- Project reviewed by late spring 2021
- **Leading to “ready for construction” milestone by June 2021**
- Site selected - to be acquired in a few weeks, permitting process well underway
- Tritium supply secured, HTS supply secure





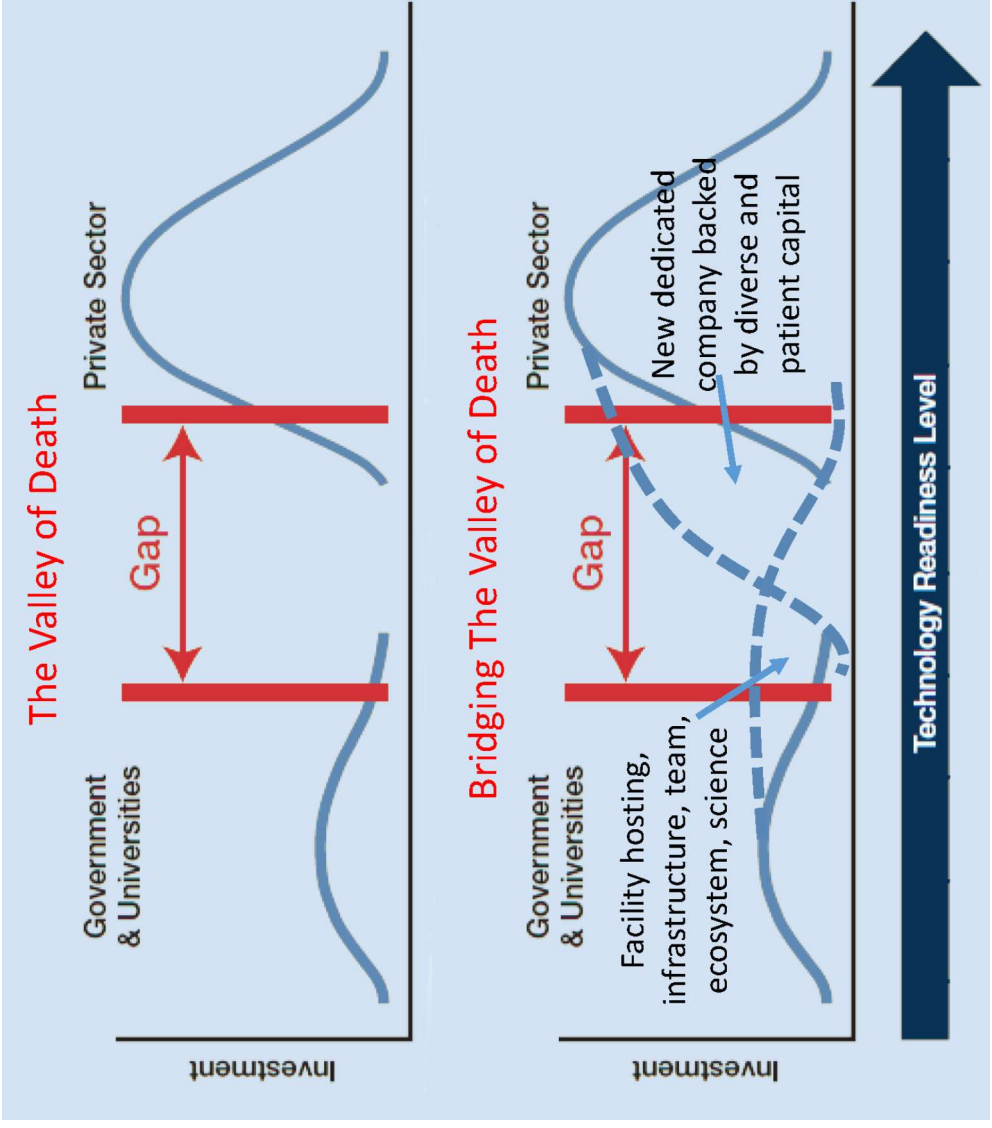
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# **SPARC Is Privately Funded**

## **The Role of Private Industry In Fusion Developm**



# Bridging the Valley of Death for Fusion



- Government Labs & Universities are basic research
- Outside of defense applications, the industry is reluctant to sponsor research as it has no application
- Private companies are reluctant to sponsor research if the time scale is long and financial returns are too low
- This is a particular problem for “Hazardous” applications
  - MIT spun out a new fusion start-up (Commonwealth Fusion Systems)
    - We’re partnering to bridge the gap
    - The university “holds on” to the technology longer, private industry picks it up



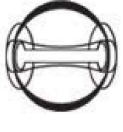


# CFS, an MIT spinout, Is Part Of A Nascent Fusion Industry



## Commonwealth Fusion Systems

- There are many companies, the list is growing
- They serve as indicators of the fusion value proposition
- Most are pursuing more speculative approaches - optimizing economics, maintainability
- Private industry/investors have a greater tolerance for risk than the USG



tokamak  
energy  
a faster way to fusion



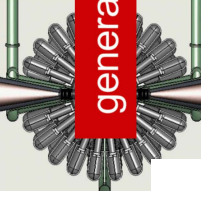
tae  
TECHNOLOGIES



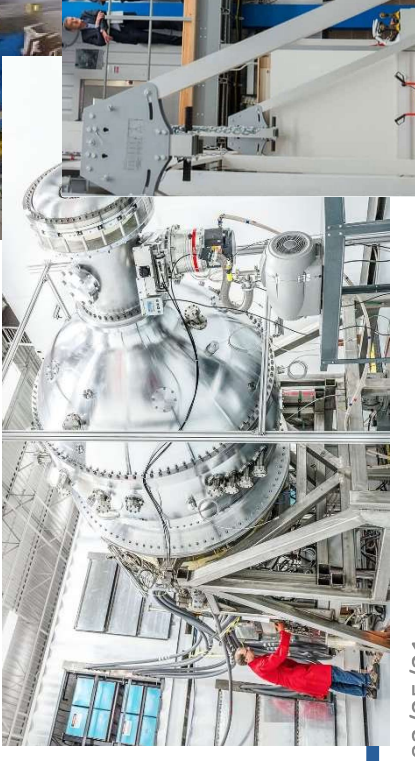
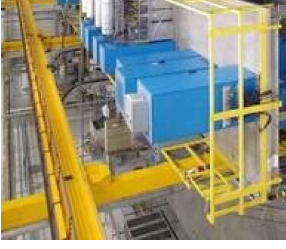
Helion Energy



PHOENIX NUCLEAR  
PARTNERING WITH TECHNOLOGY FOR THE FUTURE



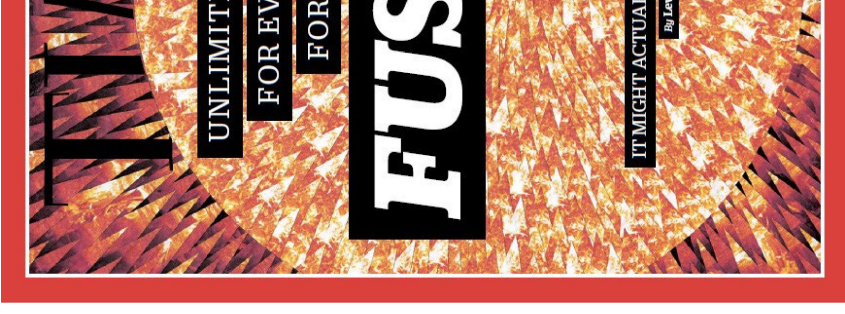
general fusion



# We think this is a good thing

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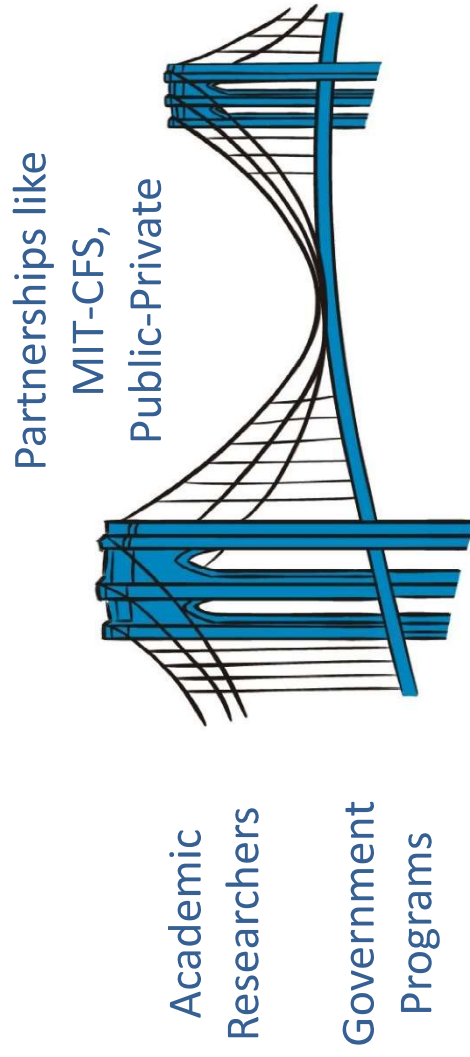
- Our basic research is being used! It is important and valued!
- We're attracting new stakeholders
- Energizing the public and adjacents
- Building momentum, building the ecosystem
- Having more players is good
- Diversity of physics approaches –fusion is too important for one architecture –not everything has to work
- Diversity of organizational approaches – fusion is too important for one team
- Diversity of tolerance to risk



# This is how new technology gets to market

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- Fusion is following a well-worn tech-development arc
  - Drugs, Aerospace, Computers, AI, Robotics, Rockets, etc
- **This is how fusion is going to get on the grid**
  - The US government doesn't build reactors, pilot plants, etc – it does basic research
  - Look to fission, fossil, ARPA-E, EERE
- How does the government program fill its role?
  - The mandate is basic research
  - To support a fusion industry



# Summary

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- **Success from our research on HTS magnets and SPARC will open up a compelling path for smaller, faster, cheaper fusion development**
  - We should have answers on the magnet technology in less than 6 months
  - Aim to demonstrate the first (controlled) net-energy fusion experiment 4 - 6 years
  - These steps will be funded by private investment
- **As a private initiative, it provides opportunities that the US program can leverage**
  - Builds on the solid foundations of decades of publicly funded research
  - Validates the goals and value of fusion research
  - Builds an industry and ecosystem which helps focus the publically funded program





