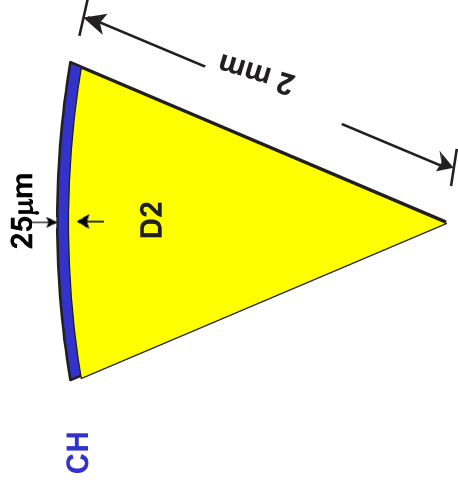
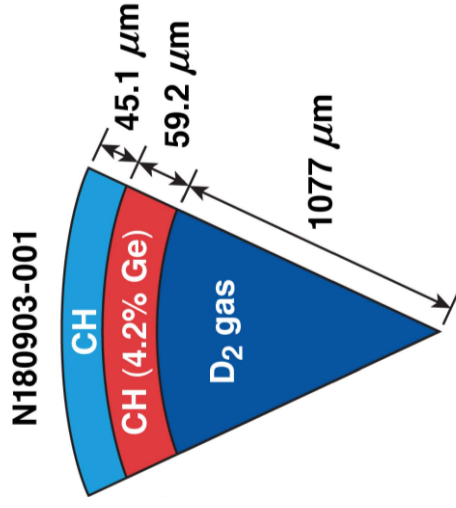


The MegaJoule Direct Drive Campaign: NIF Experiments on the Pathway to MJ Yield

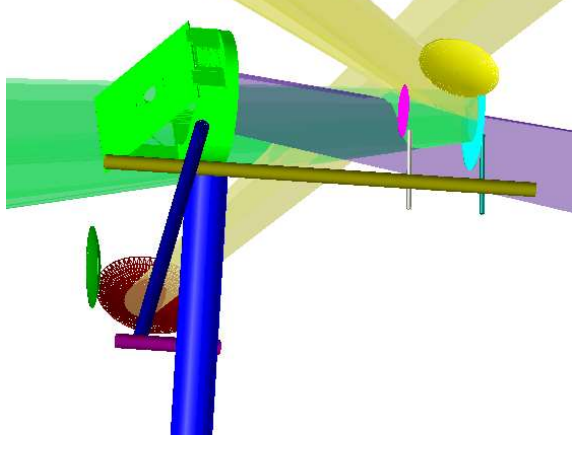
Energy coupling/symmetry



Hot electron preheat



Laser imprint



Michael Rosenberg
University of Rochester
Laboratory for Laser Energetics

HE

Current MJ-scale experiments lay the groundwork for high-yield direct-drive implosions on the NIF in the 2020's

- Extrapolation of direct-drive implosion performance from OMEGA to ignition scale on dedicated physics experiments at NIF scale
- The MegaJoule Direct Drive (MJDD) campaign on NIF includes three primary physics experiments:
 - 1) Hot electron preheat has been measured at close to tolerable levels, partially n using mid-Z layers, and shown to originate from stimulated Raman scattering
 - 2) Implosion symmetry and energy coupling (including cross-beam energy transfer) been controlled with wavelength detuning, contoured shells, and beam pointing
 - 3) Laser imprint experiments have produced x-ray radiography data to benchmark of imprint-related instabilities and partial mitigation with multi-FM SSD or high
- Current NIF capabilities limit high-performance polar-direct drive (PDD) implosion convergence exploding pushers

A staged set of proposed facility improvements will allow for increased convergence

Collaborators

A. Shvydky, A. A. Solodov, J. Marozas, P. McKenty, P. B. Radha, W. Theobald, J. Peebles, D. P. Turnbull,
Follett, R. Epstein, R. Betti, A. V. Maximov, T. J. B. Collins, K. Anderson, M. Bedzyk, V. N. Goncharov, J.
Froula, and S. P. Regan
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M. Hohenberger, P. Michel, A. R. Christopherson, C. Yeamans, G. Kemp, H. Whitley, B. Bachmann, G. Hall,
Kerr, S. Khan, L. Divol, J. Hernandez, G. Swadling, N. Lemos, E. Tubman, J. S. Ross, T. Chapman, J.
Lawrence Livermore National Laboratory

M. Karasik
Naval Research Laboratory

L. Antonelli
University of York

C. Krauland, C. Shuldberg, M. Farrell
General Atomics

R. Scott and K. Glize
Rutherford Appleton Laboratory

J. F. Myatt
University of Alberta

A. Colaitis
CELIA

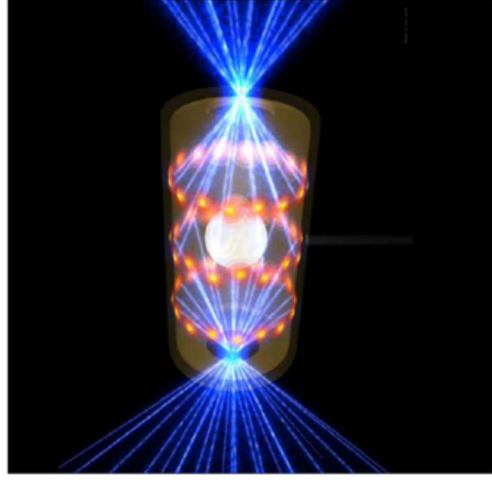
Outline

- **Laser direct-drive inertial confinement fusion**
 - **Scaling and motivation for NIF experiments**
- **MJDD experiments on NIF**
 - **LPI/hot electron preheat**
 - **Laser energy coupling/CBET/PDD symmetry**
 - **Laser imprint**
- **Current high-yield PDD implosions and pathway to MJ yield**

Motivation

Laser direct drive is one of three main approaches to achieving in confinement fusion ignition and high yield

Laser indirect drive

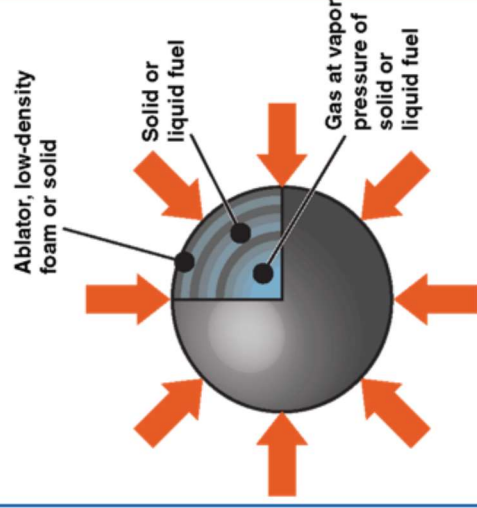


LLNL lead laboratory
(LANL)

Facility

- NIF — OMEGA

Laser direct drive

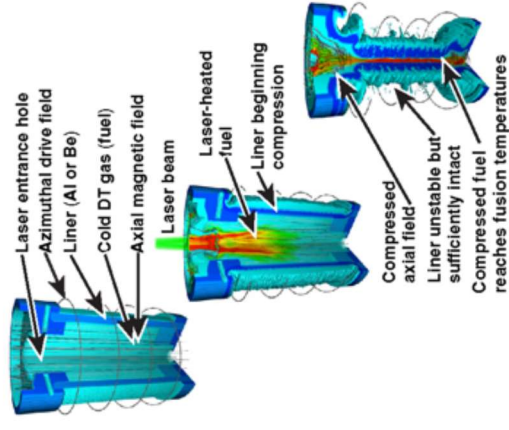


LLE lead laboratory
(NRL, LLNL, LANL)

Facility

- OMEGA
— NIF
— NIKE

Pulsed-power magnetized fusion



SNL lead laboratory
(LLNL, LANL, LLE)

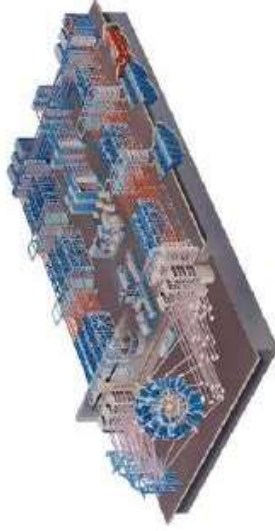
Facility

- Z — Omega
— NIF

12415g

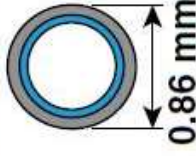
Most direct-drive experiments to produce high-performance (albeit $\sim 1/4$ of ignition scale) implosions are done on OMEGA, which is designed for LDD experiments

OMEGA



Scale 1:70
in energy

OMEGA 26 kJ

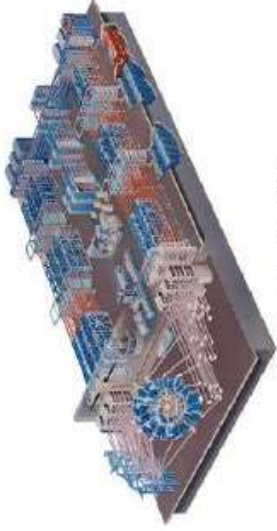


OMEGA Advantages for LDD

- Symmetric illumination
- Circular beam spots (ideal for symmetric illumination)
- State-of-the-art beam smoothing (2D smoothing by spectral dispersion, polarization smoothing)
- Cryogenic capability for LDD

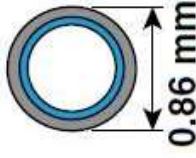
NIF has laser energy (1.9 MJ) closer to ignition scale, but is not (yet) optimized for high-performance direct-drive implosions

OMEGA



Scale 1:70
in energy

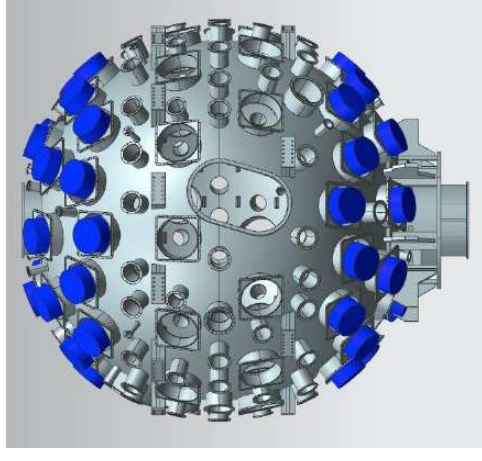
OMEGA 26 kJ



OMEGA Advantages for LDD

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NIF



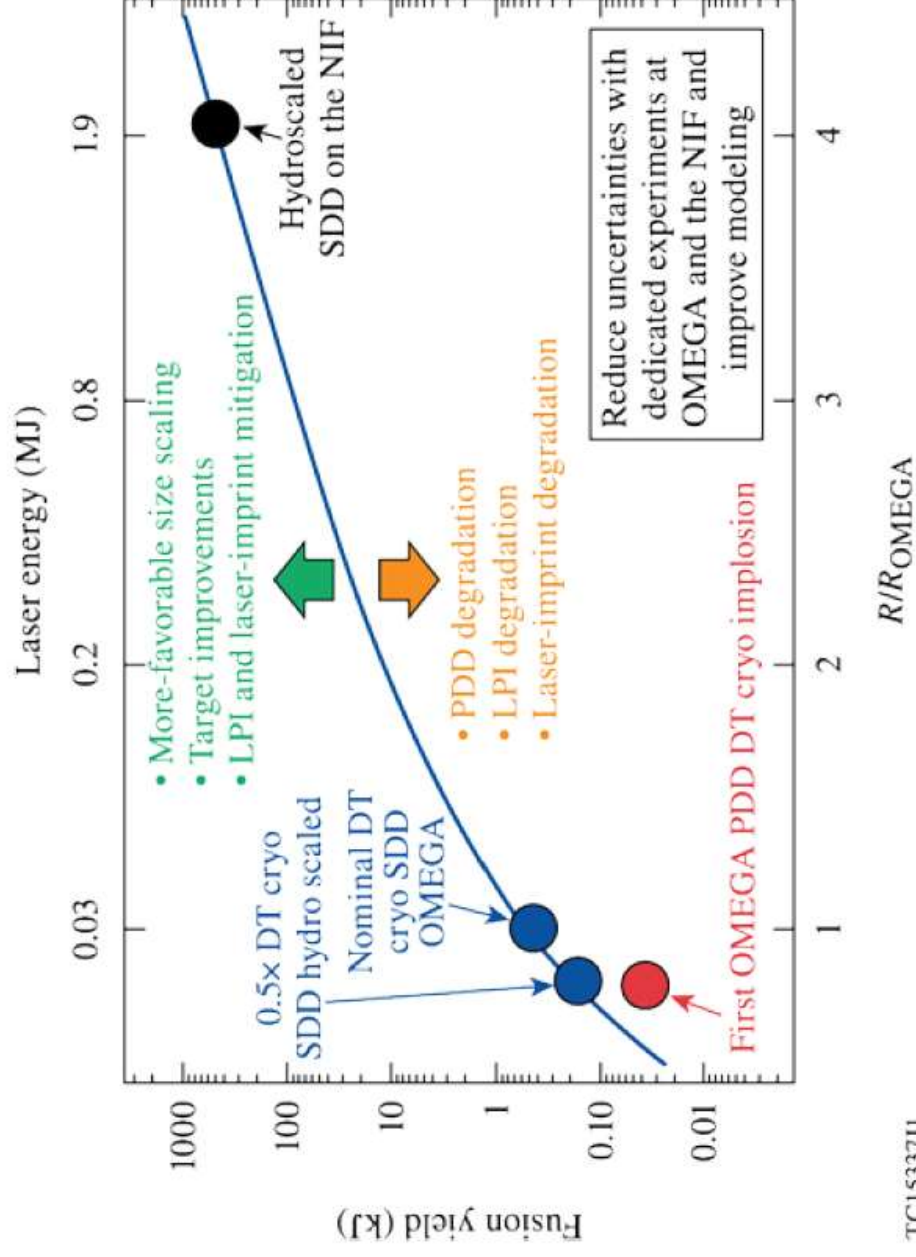
NIF optimized for indirect-drive

- Polar illumination
- Beams must be defocused for LDD
- Limited beam smoothing
- No LDD cryo capability

Despite limitations, NIF can be utilized to learn about direct drive physics at ignition-relevant energies now, and with some improvements to the facility, work towards high-performance indirect-drive implosions

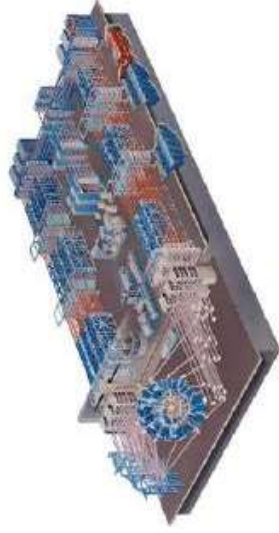
Motivation

Current OMEGA cryogenic implosion yield extrapolates to ~570 kJ at 2 MJ laser drive assuming hydrodynamic scaling is valid



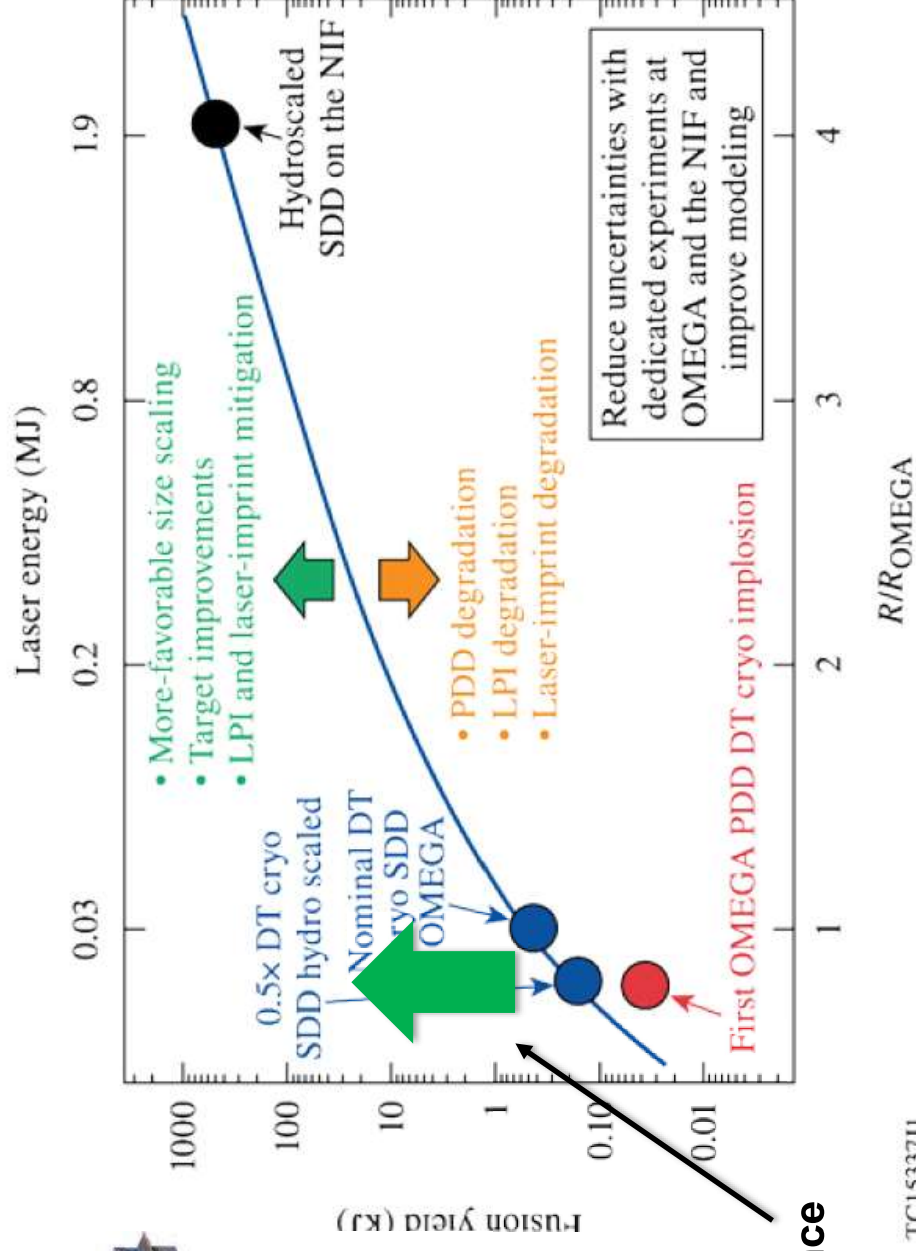
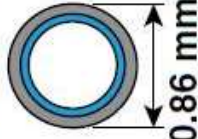
Motivation

In this context, there are two major thrusts of the direct-drive ICF effort, encompassing experiments on both OMEGA and NIF



Scale 1:70
in energy

OMEGA 26 kJ

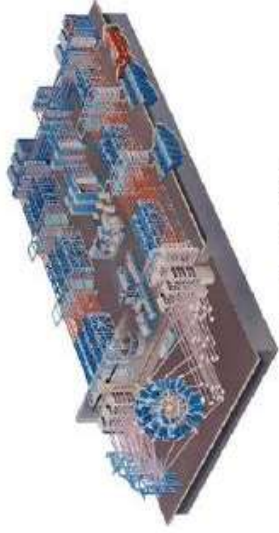


1) Improve performance on OMEGA*

* V. Gopaldaswamy et al. *Nature* 565, 581 (2019)

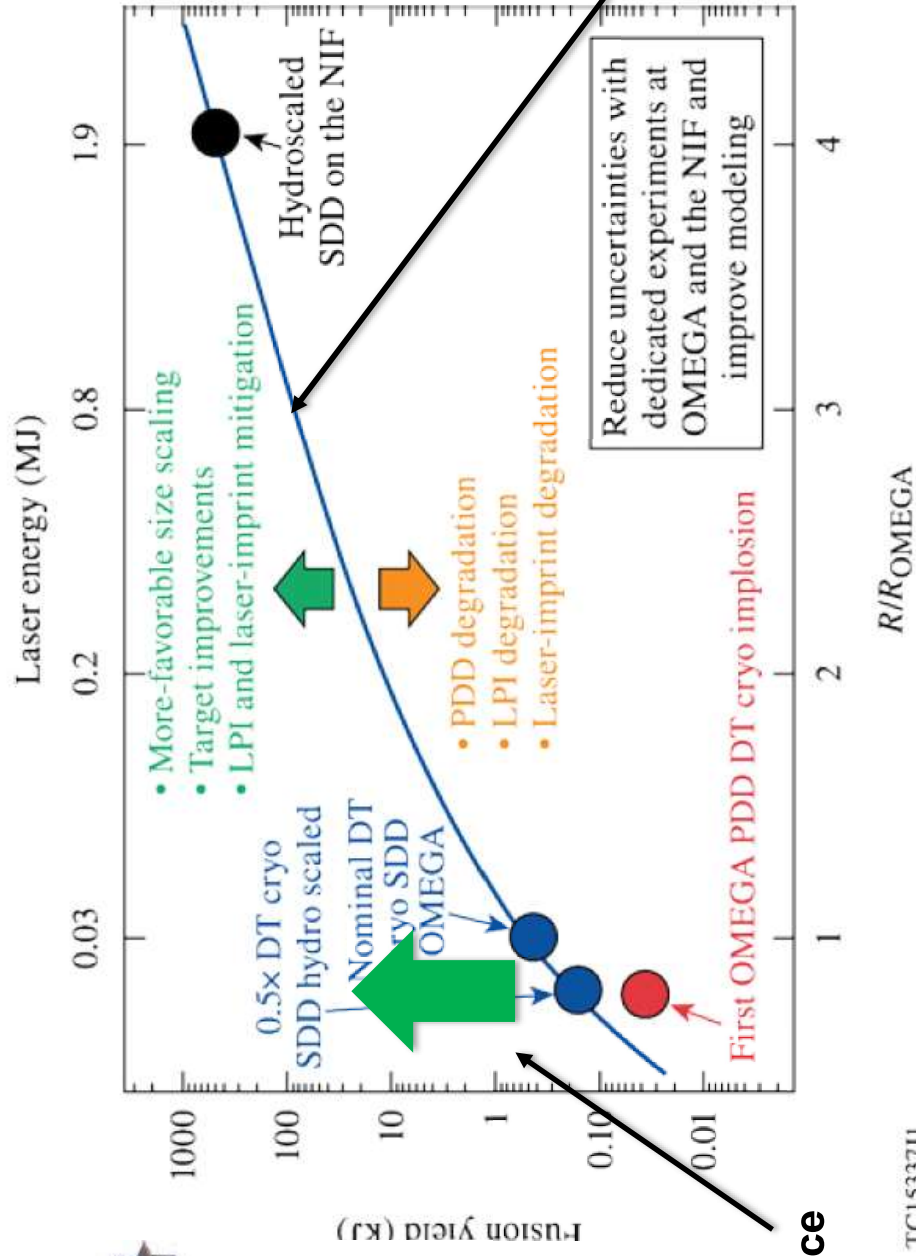
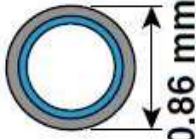
Motivation

In this context, there are two major thrusts of the direct-drive ICF effort, encompassing experiments on both OMEGA and NIF



Scale 1:70
in energy

OMEGA 26 kJ



1) Improve performance on OMEGA*

2) Focused assessment/valid MJDD campaigns experiments necessarily scaling: Energy preheat, implosions and

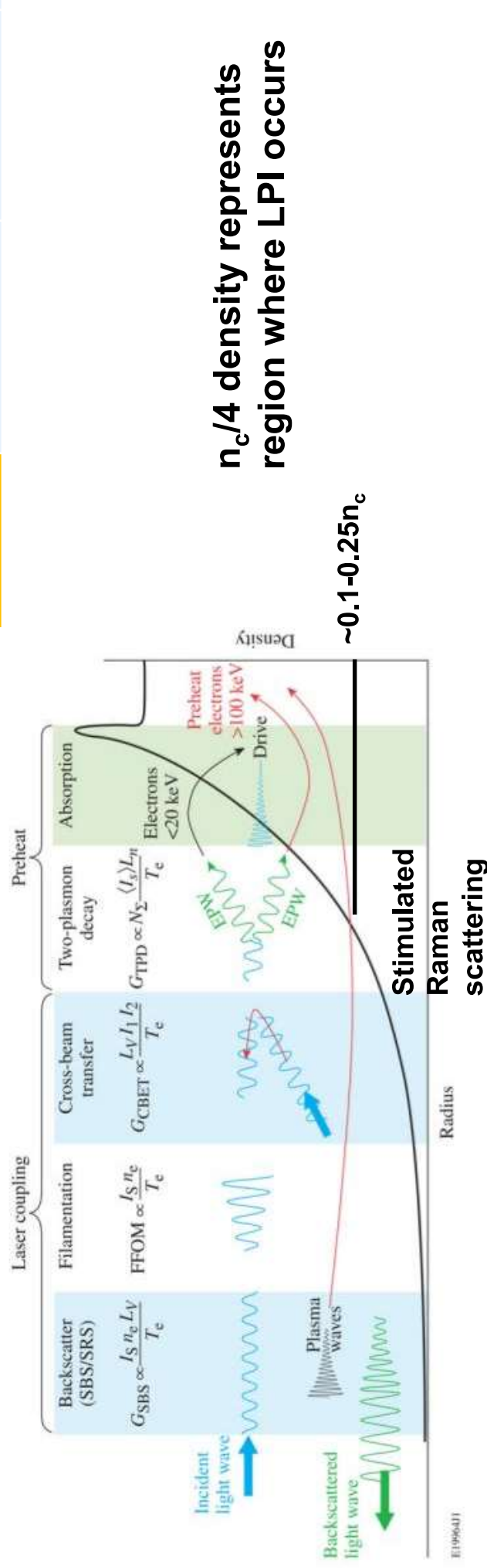
* V. Gopalaswamy et al. Nature 565, 581 (2019)

Motivation

NIF experiments achieve ignition-relevant coronal plasma conditions are inaccessible in OMEGA implosions

2D DRACO simulated plasma conditions at

	NIF ignition scale	NIF planar experiments	NIF sub-scale implosions
L_n (μm)	600	400-700	400
T_e (keV)	3.5 to 5	3 to 5	3.2



$n_c/4$ density represents region where LPI occurs

E1996411

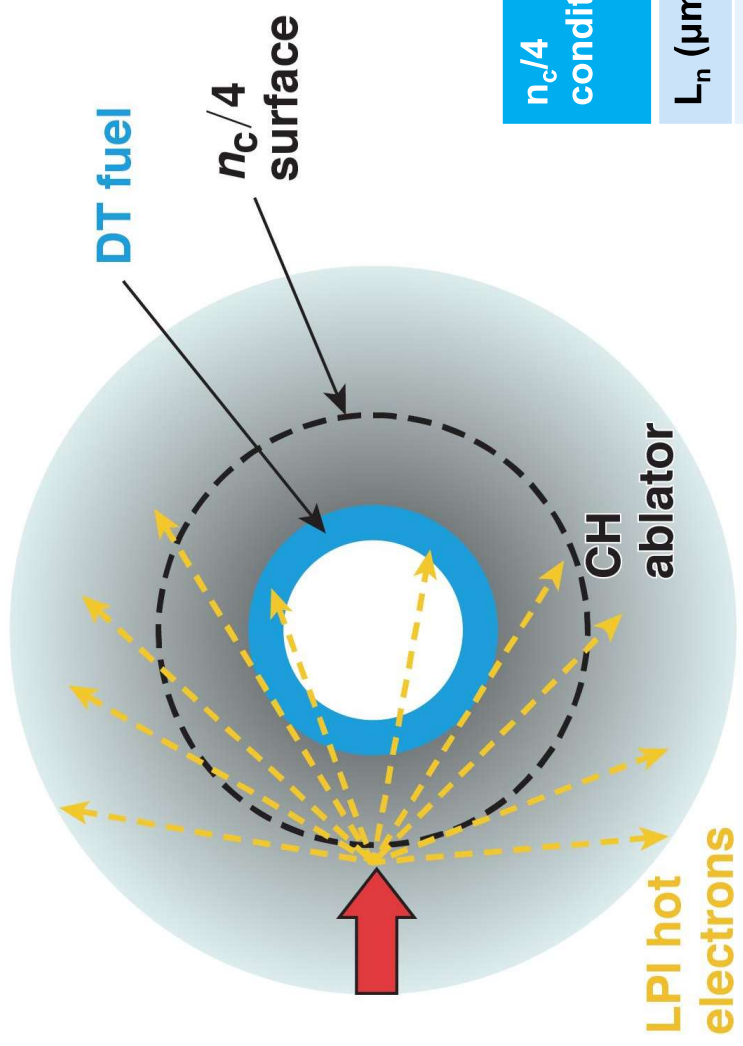
Outline

- Laser direct-drive inertial confinement fusion
 - Scaling and motivation for NIF experiments
- **MJDD experiments on NIF**
 - LPI/hot electron preheat
 - **Laser energy coupling/CBET/PDD symmetry**
 - **Laser imprint**
- Current high-yield PDD implosions and pathway to MJ yield

LPI and hot electron preheat has been studied in both planar and spherical geometries to determine preheat levels and assess mitigation at ignition-relevant scale lengths

Preheat in cryo implosions

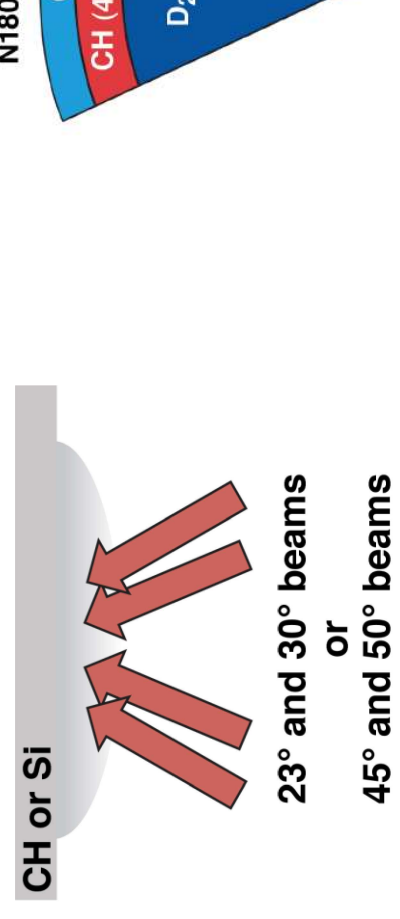
Direct-drive implosion



NIF experimental platforms

Planar platform (to study LPI mechanisms and hot electron production) coupling to

Spherical platform (to study)



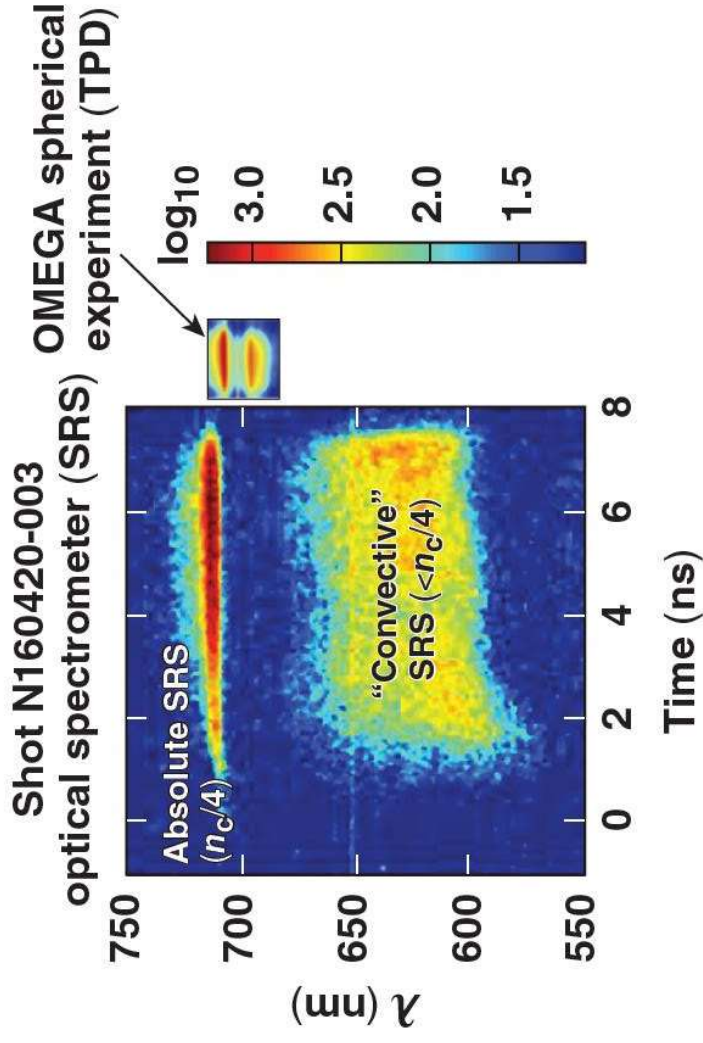
n _c /4 conditions	NIF ignition scale	NIF planar experiments	NIF spherical experiments
L _n (μm)	600	400-700	400
T _e (keV)	3.5 to 5	3 to 5	3.2
I _L (W/cm ²)	(6-10)x10 ¹⁴	(6-15)x10 ¹⁴	3-6x10 ¹⁴

Designer: A. Solodov

Planar LPI experiments established the predominance of stimulated Raman scattering (SRS) as a hot electron source

NIF: $L_n = 525 \mu\text{m}$
 $T_e = 4.5 \text{ keV}$

OMEGA:* $L_n = 150 \mu\text{m}$
 $T_e = 2.8 \text{ keV}$

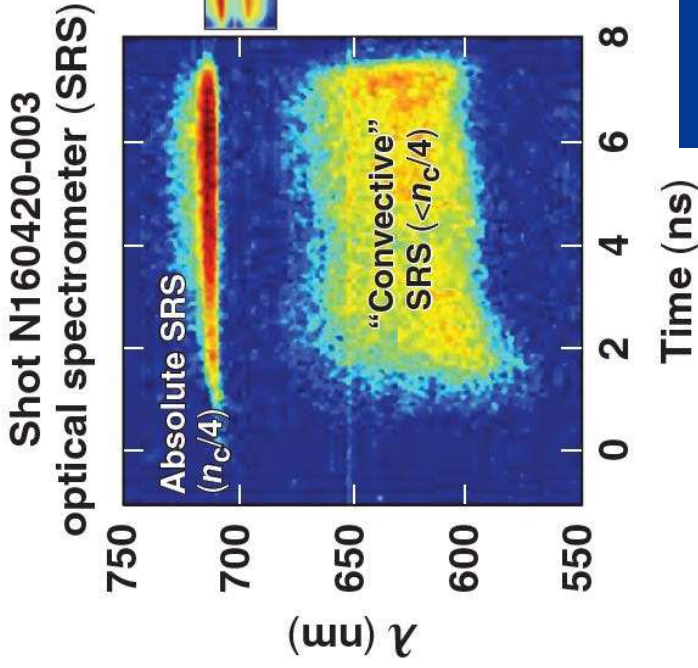


M. J. Rosenberg et al., *Phys. Rev. Lett.* 120, 055001 (2018)

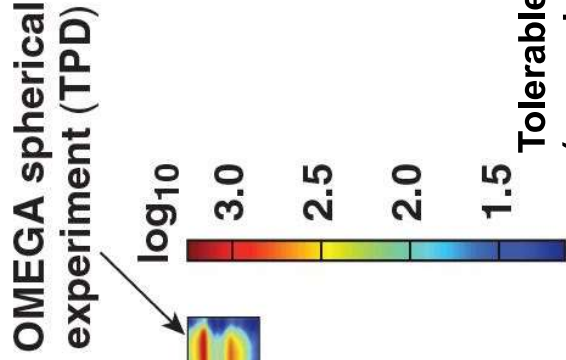
M. J. Rosenberg et al., *Phys. Plasmas* 27, 042705 (2020)

Planar LPI experiments established the predominance of stimulated Raman scattering (SRS) as a hot electron source, the intensity scaling of preheat, and Si mitigation

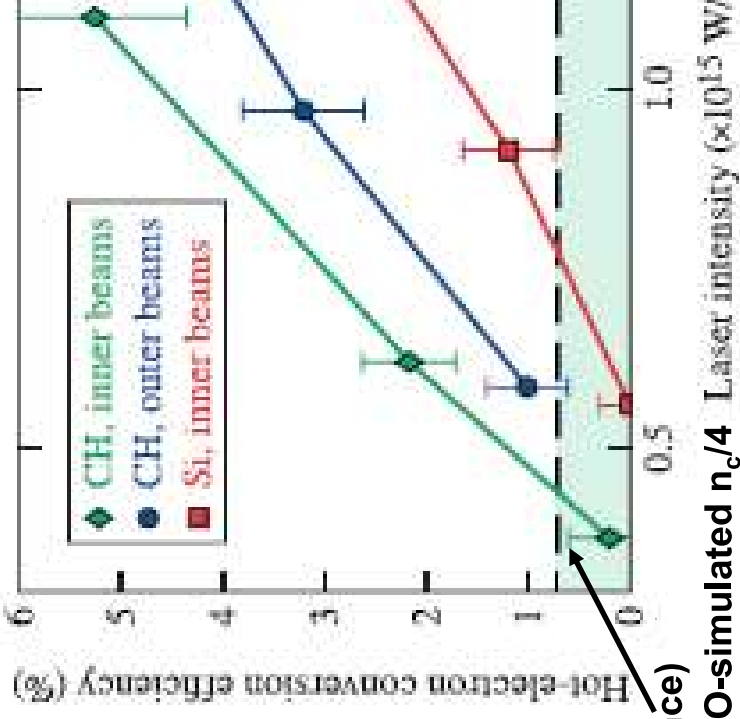
NIF: $L_n = 525 \mu\text{m}$
 $T_e = 4.5 \text{ keV}$



OMEGA: $L_n = 150 \mu\text{m}$
 $T_e = 2.8 \text{ keV}$



Tolerable preheat level
 (assuming wide divergence)



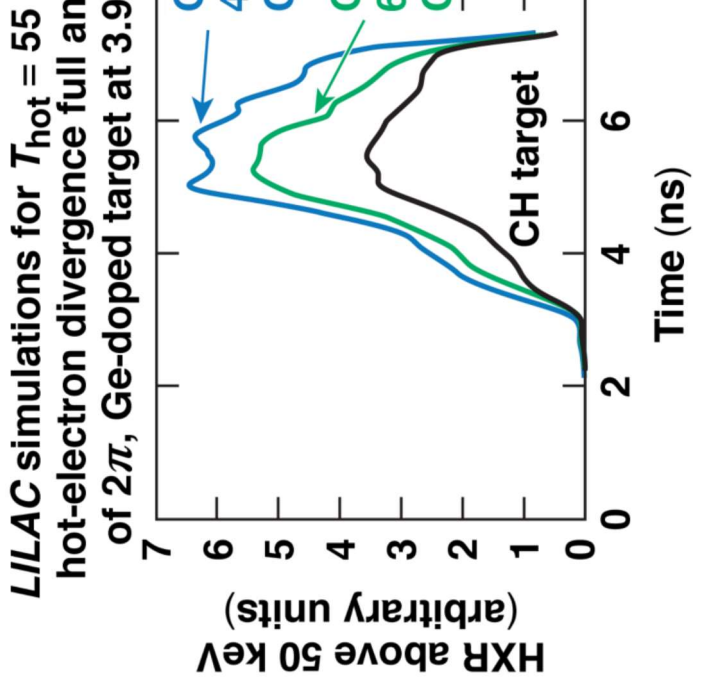
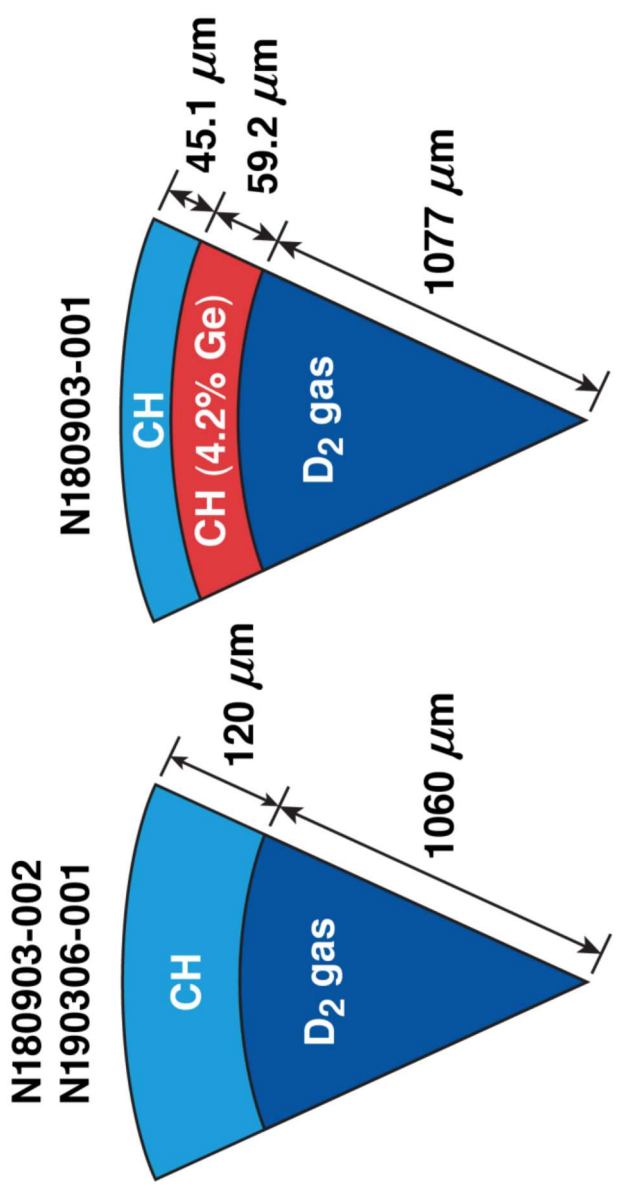
Preheat in implosions depends not only on hot electron production, but divergence/coupling to the inner portion of the shell

M. J. Rosenberg et al., Phys. Rev. Lett. 120, 055001 (2018)
 M. J. Rosenberg et al., Phys. Plasmas 27, 042705 (2020)

A. A. Solodov et al., Phys. Plasmas

A PDD implosion platform, using Ge-doped layers, was adapted for OMEGA* to diagnose preheat in the inner portion of the shell

NIF target designs

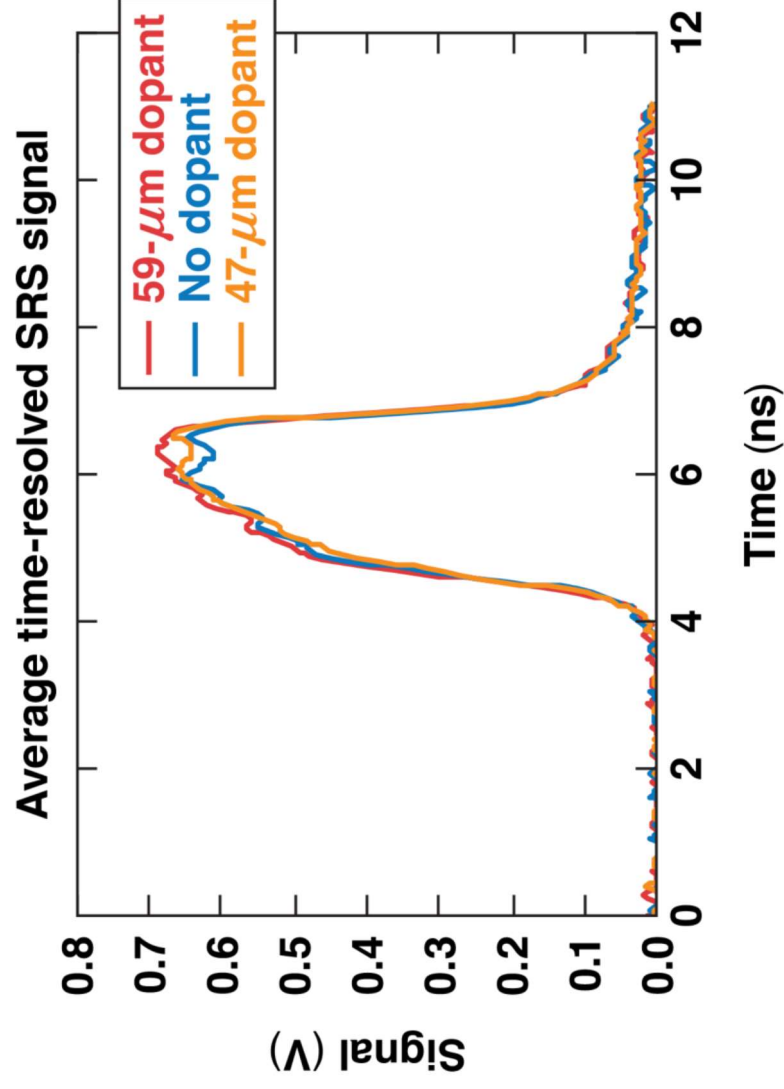


Design: A. Solodov
E29257

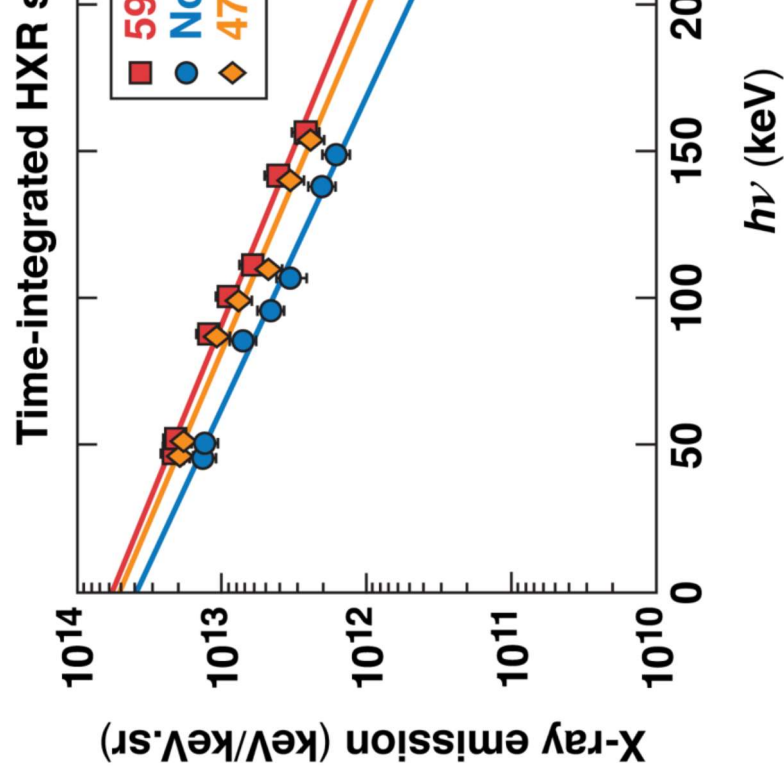
For an identical laser drive and identical hot electron source, the difference in hard x-rays \propto hot electron energy deposited in Ge-doped layer

*Platform based on A. Christopherson, et al. (sub)

Hard x-ray (HXR) emission on NIF shows the expected variation with Ge layer thickness, with identical LPI



Identical LPI/hot e⁻ source

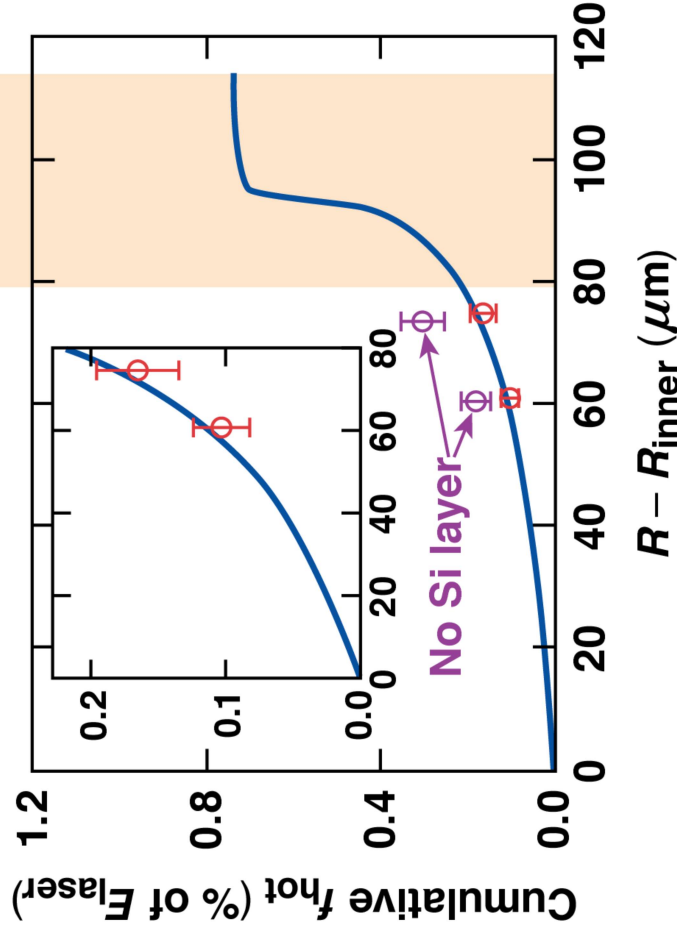
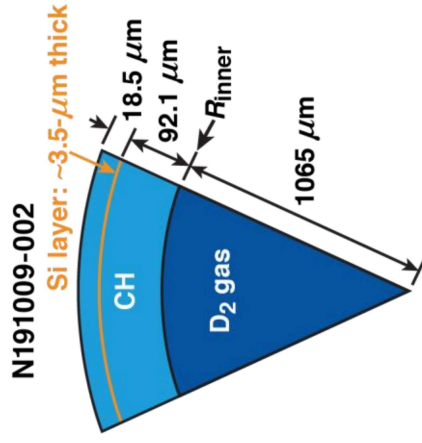


Different HXR emission

E27877a

Hot electron preheat in NIF implosions is inferred to be $\sim 0.2\%$ of I over the inner $\sim 80\%$ of unablated shell, and $\sim 2\times$ lower with Si layer

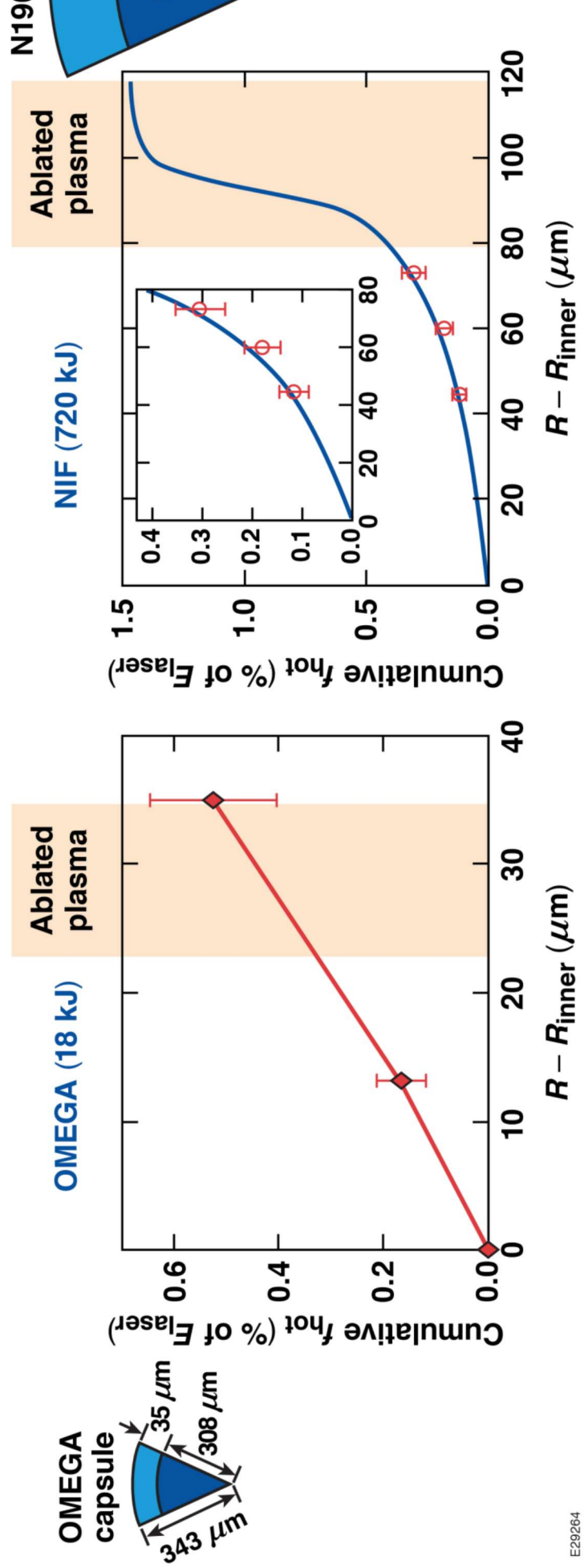
Incident intensity = 10^{15} W/cm²



Analysis: A. Solodov

This level of preheat is close to $\sim 0.15\%$ limit for direct-drive ignition des

Hydrodynamically-scaled version of this experiment on OMEGA with similar fraction of laser energy coupled to the inner shell as preheated

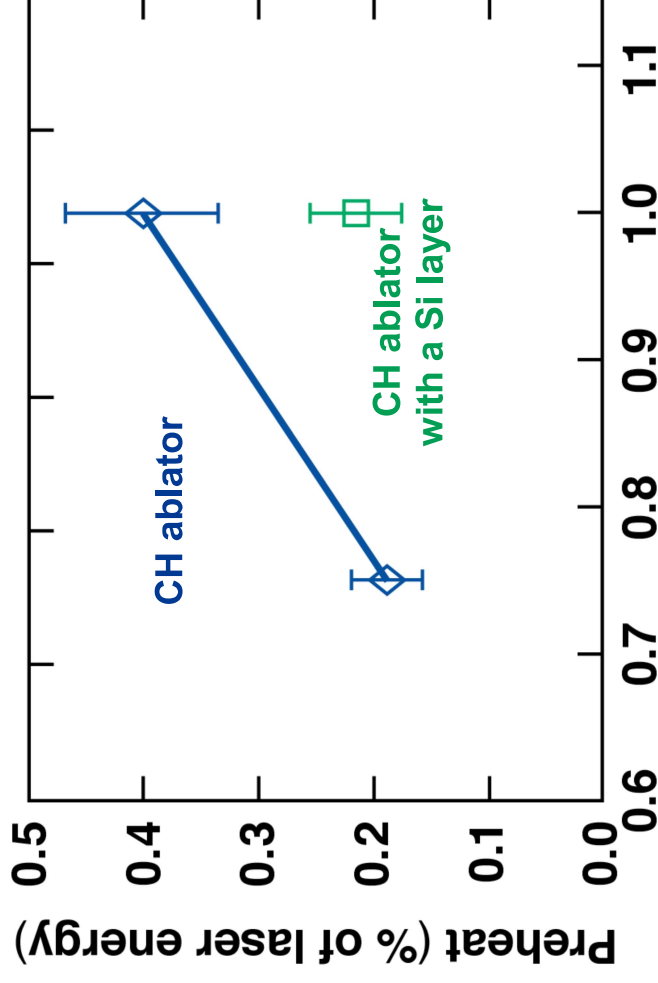


E29264

These results support validity of hydro-scaling in warm implosions, though hot electron attenuation at the outer ablator is important and will have to be accounted for in cryogenic implosions with ablaters

The goal of NIF experiments is to determine the parameter regime (e.g. intensity) that produces acceptably low preheat

Hot-electron energy deposited in the unablated shell



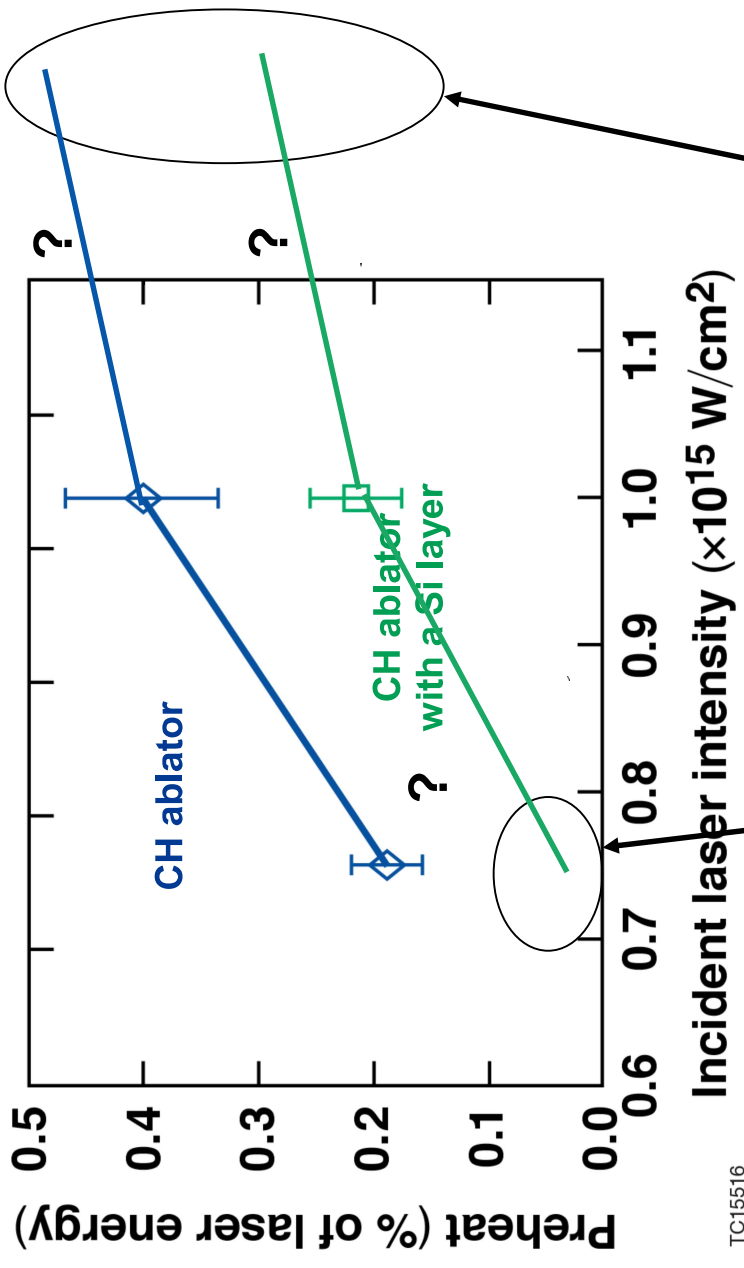
Analysis: A. Solodov

TC15516

Incident laser intensity ($\times 10^{15}$ W/cm²)

The goal of NIF experiments is to determine the parameter regime (e.g. intensity) that produces acceptably low preheat

Hot-electron energy deposited in the unablated shell



Analysis: A. Solodov TC15516

Goal of NIF experiments this year is to calculate preheat vs. intensity with and with

These data can be used to estimate likely preheat levels in ignition scale cryogenic at 10^{15} W/cm² on-target intensity

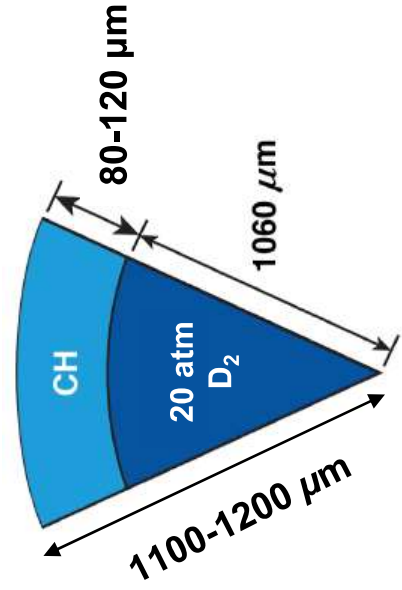
	Multiplier	Preheat (% of laser)
Preheat into inner 80% of unablated shell in warm subscale NIF implosion	-	~0.2%
Increase scale length to full scale*	~1.5-2	~0.3-0.4%
Increase convergence ratio at end of pulse	~0.4-0.8	~0.15-0.3%
DT shell and some DT in ablator	~1-1.8	~0.15-0.5%
Improve beam smoothing	~0.8	~0.12-0.4%
Si layer	~0.5	~ 0.06-0.2%
Total		~ 0.1-0.2%?

On-target intensity close to 10^{15} W/cm² looks to be OK (so far) for preheat – further experiments are planned to study effect of convergence, mitigation with Si dopant, and ignition-scale capsules

Note: current preheat results are near-“worst case scenario” given poor beam smoothing

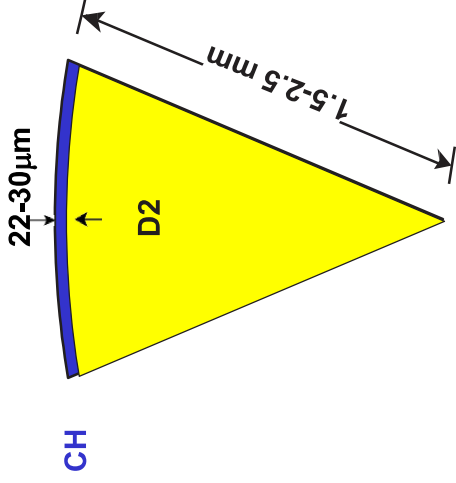
Energy coupling/symmetry experiments have been conducted in spherical-geometry platforms at a variety of convergence ratios

PDD implosion platform (CBET/ $\Delta\lambda$)



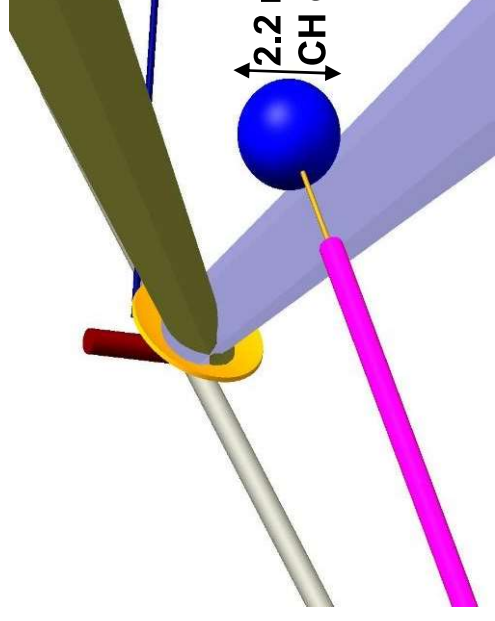
CR ~12

“Exploding pusher” platform



CR ~7

Solid sphere radiography platform



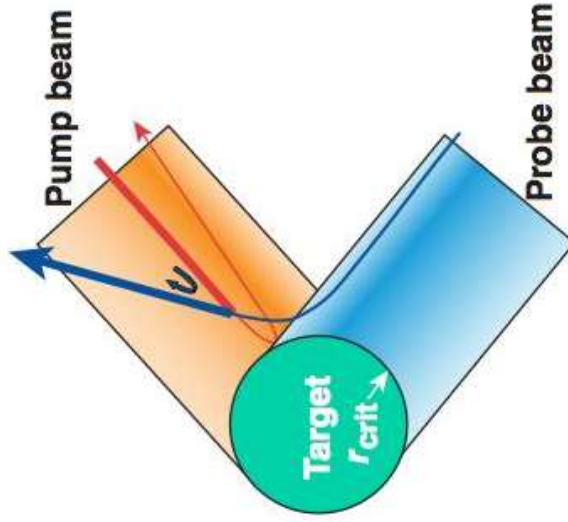
CR =1

Integrated implosion experiment

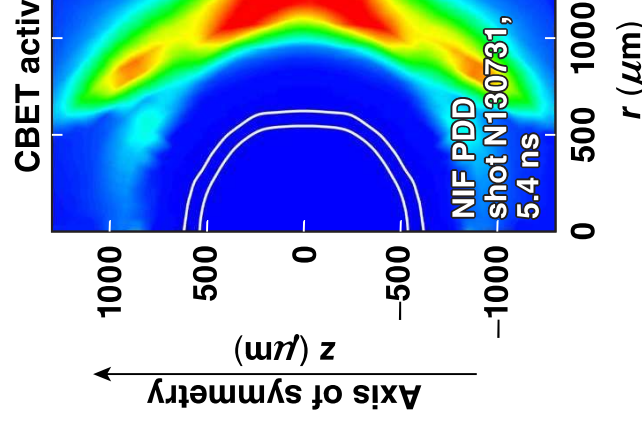
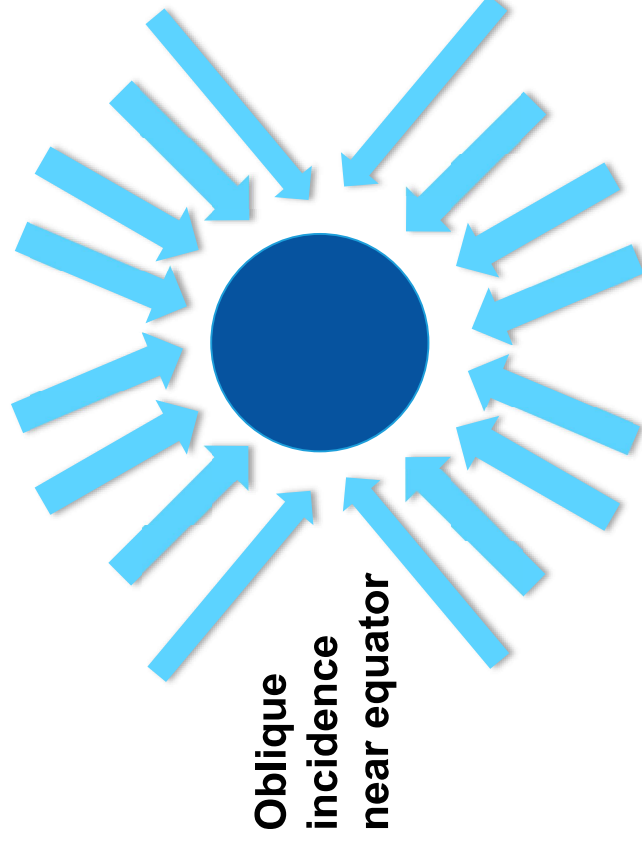
Focused coupling experiment

In polar direct drive (PDD) implosions, cross-beam energy transfer (CBET) is a primary energy loss mechanism and occurs predominantly near the equator

CBET is the transfer of laser energy from incoming (pump) rays to outgoing (probe) rays



PDD illumination geometry

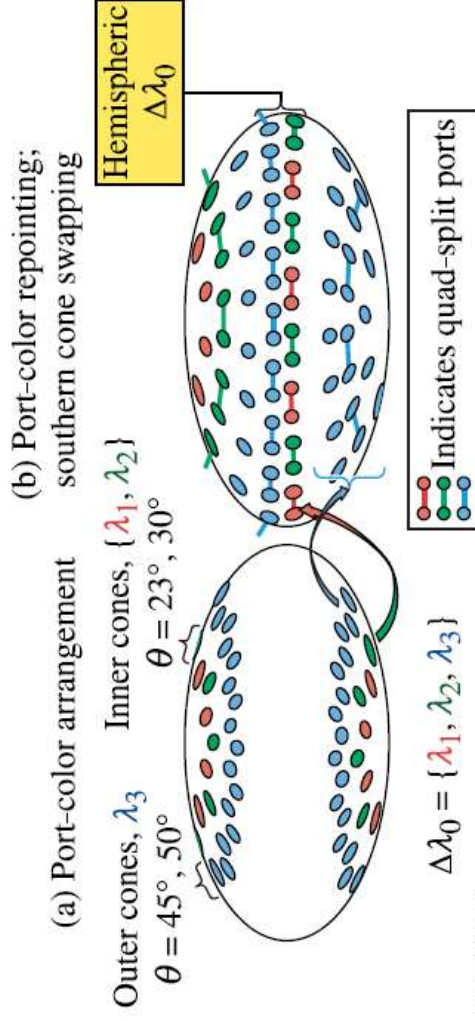


Wavelength detuning ($\Delta\lambda$) between interacting beams is one option for CBET

P. B. Radha et al., *Phys. Plasma*

PDD implosions used cone-swapping to produce hemispheric wavelength detuning across the equator

Cone-swapping for $\Delta\lambda$ across the equator

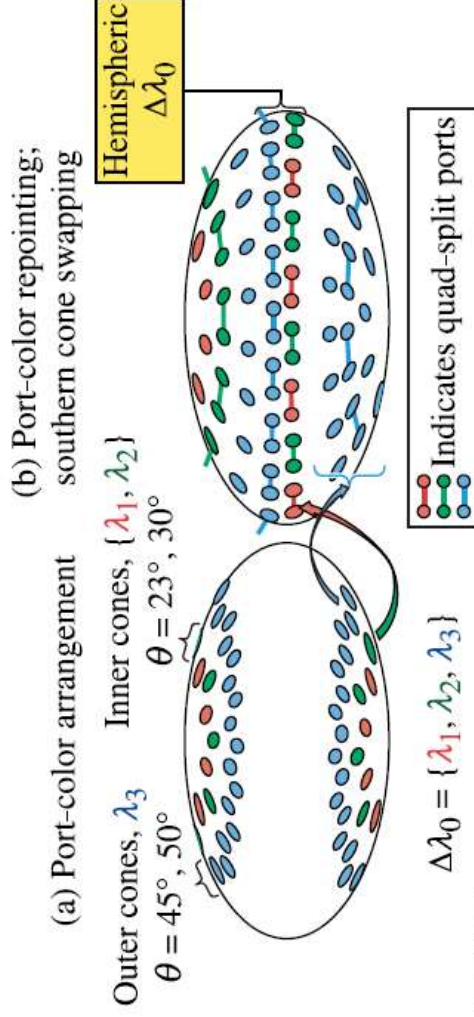


TC11736J1

Design and simulations: J. Marozas
X-ray analysis: M. Hohenberger and D. Turnbull

PDD implosions with hemispheric ± 2.3 Å (UV) demonstrated partial mitigation of CBET, as predicted by 2D DRACO simulations

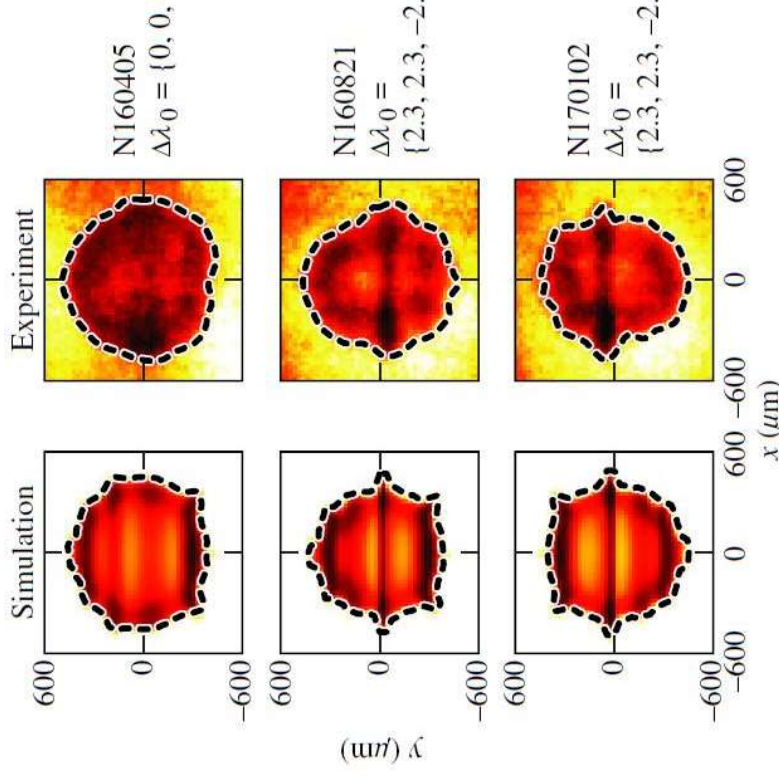
Cone-swapping for $\Delta\lambda$ across the equator



TC11736J1

Design and simulations: J. Marozas
X-ray analysis: M. Hohenberger and D. Turnbull

X-ray radiographs

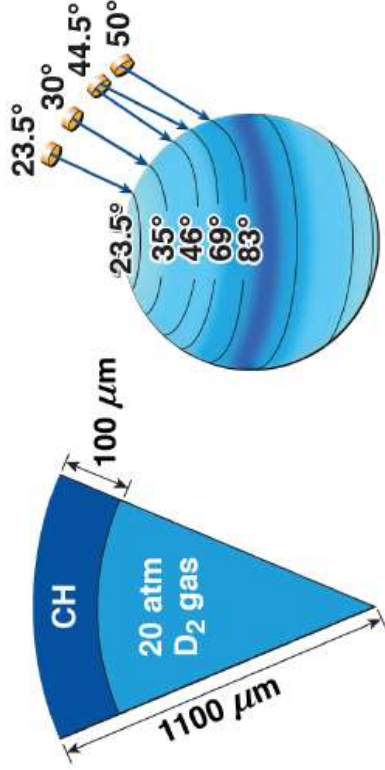


- ± 2.3 Å (UV) is current NIF limit – need $> +3$ Å, ideally ± 6 Å for greatly enhanced energy
- Flexible color/port-mapping will be implemented in 2022 to avoid cone-swapping and CBET

J. A. Marozas et al., *Phys. Rev. Lett.* 128, 155101 (2022)

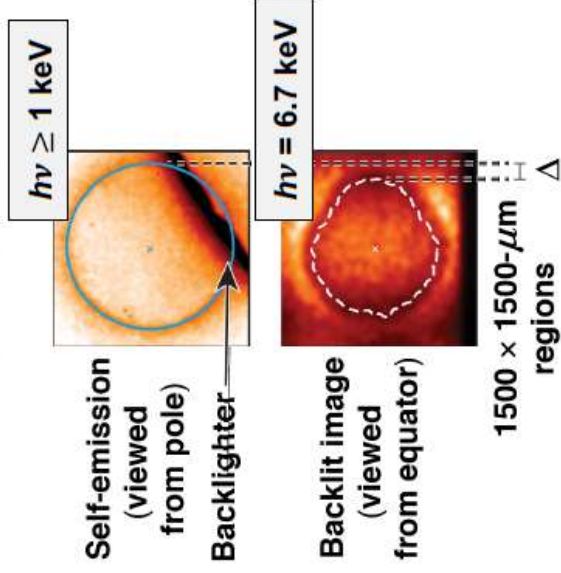
One problem for implosions is that we have been unable to simultaneously have the ablation front and inner shell trajectories – shell puffing up due to im

NIF PDD implosion
(0.65 MJ, 1.2×10^{15} W/cm²)

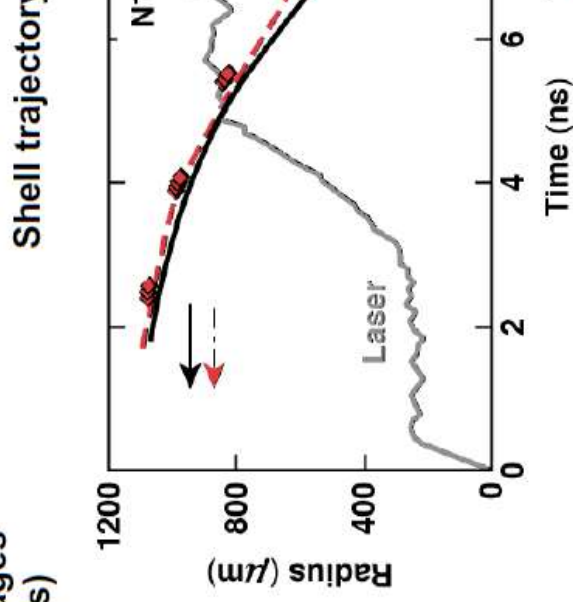


TC11724d

Measured gated x-ray images
(dx = 30 μm, dt = 100 ps)



TC11725e



$$E_{\min} \propto V_{\text{imp}}^{-6}$$

Backlit image: V_{imp} matches within 1%

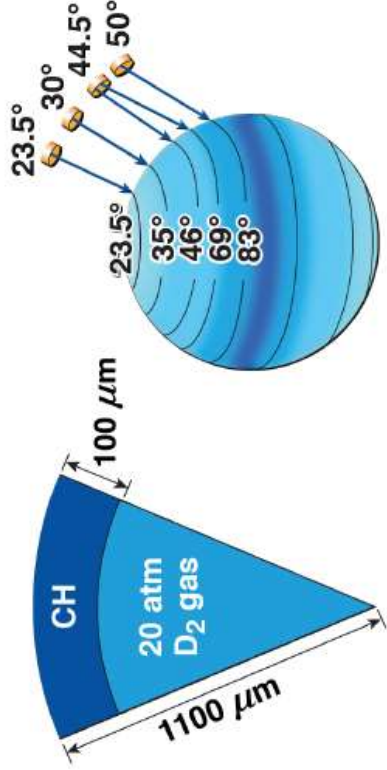
Self-emission image: V_{imp} overpredicted by 9%, attributed to imprint and Rayleigh-Taylor

Concern about imprint has led our energy coupling campaign to lower convergence

P. B. Radha et al., *Phys. Plasma*

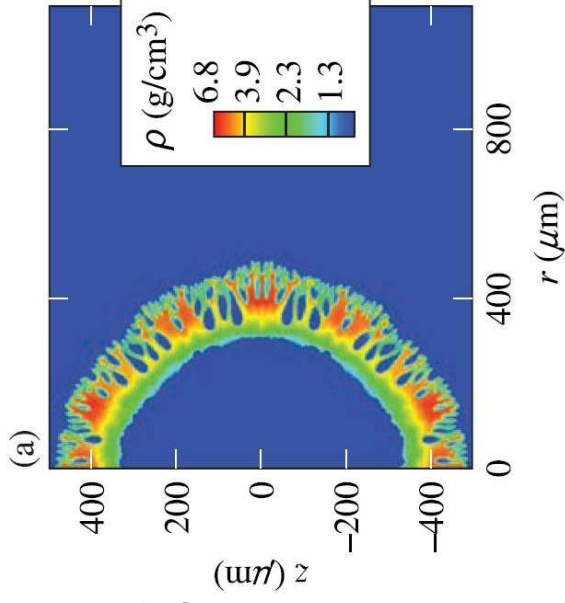
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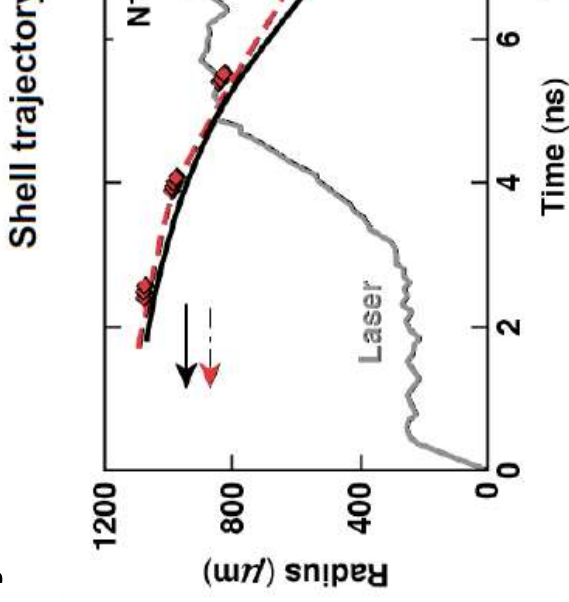


TC11724d

2D DRACO simulated density
(with laser imprint)



TC11725e



$$E_{\min} \propto V_{\text{imp}}^{-6}$$

Self-emission image: V_{imp} overpredicted by 9%, attributed to imprint and Rayleigh-Taylor

Backlit image: V_{imp} matches within 1%

Concern about imprint has led our energy coupling campaign to lower convergence

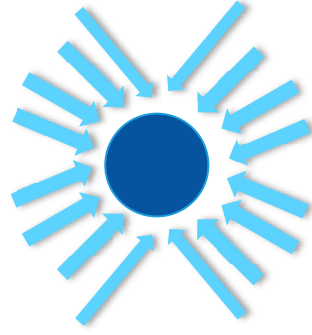
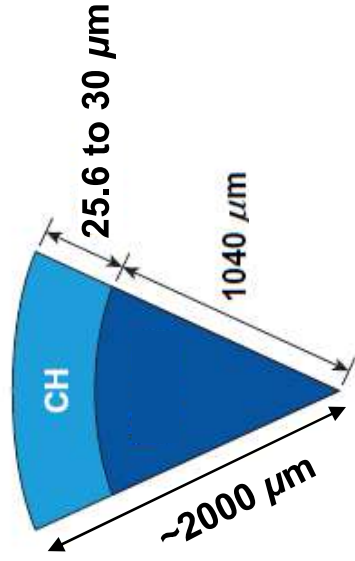
P. B. Radha et al., *Phys. Plasma*

Lower convergence implosions, less sensitive to imprint, are now being assessed/improve energy coupling and symmetry

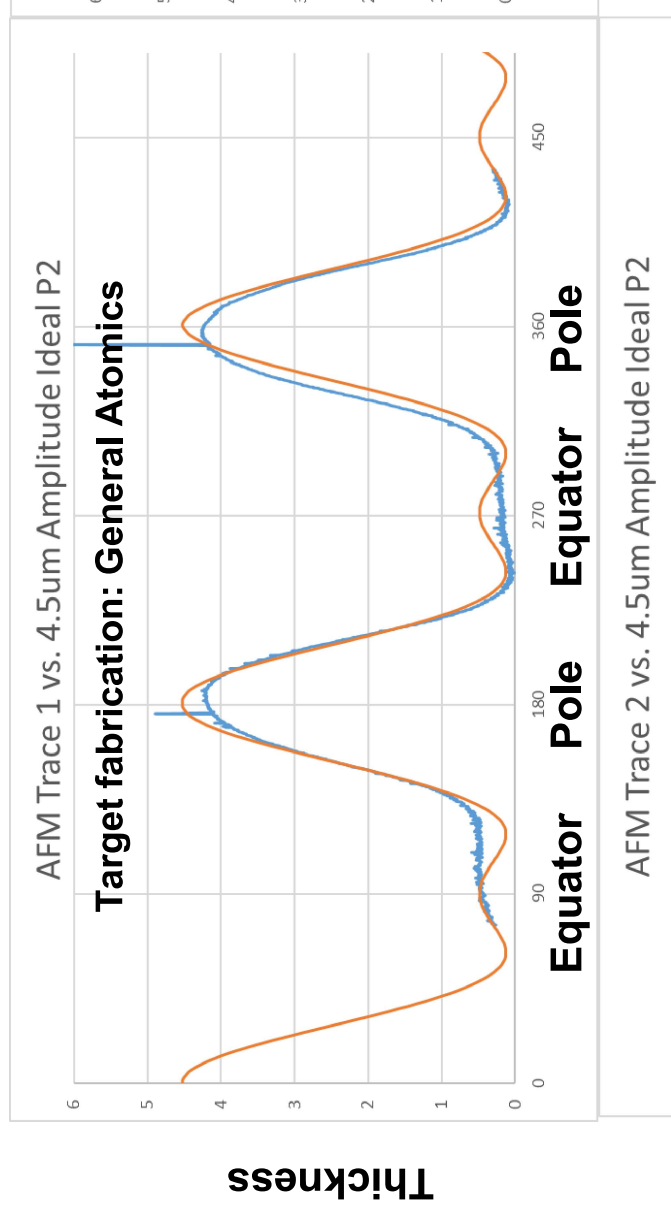
Parameter	“Standard” implosions	“Exploding pushed”
Shell diameter	2.3 mm	3-5 mm
Shell thickness	120 μm	25 μm
Primary implosion mechanism	Ablation/compression	Shock
Convergence ratio	~ 12	~ 7
Physics issues	Laser energy coupling Hydro instabilities (e.g. imprint-seeded RT)	Laser energy coupling [less susceptible to instabilities]

Exploding pusher experiments studied shell contouring as a means to implosion symmetry and improve energy coupling along the equator

800 kJ, 280 TW laser drive



Shell contoured designed based on 2D DRACO simulations of previous experiment



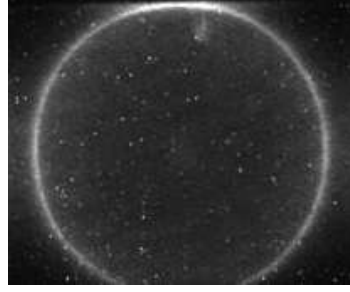
Shell thicker near the poles,
thinner near the equator

Design: P. Mc

The contoured shell eliminated a 5% in-flight P2, in agreement with pos simulations

X-ray self-emission images

t~2 ns

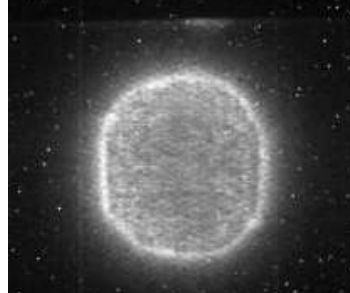


Equatorial (90-124) view

t~3 ns



t~4 ns

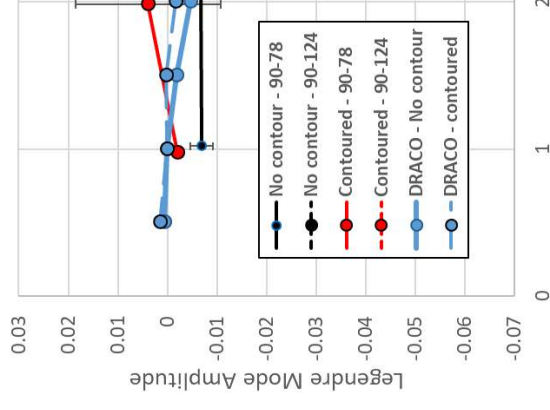


N200715-001: without contour

N201021-002: contoured shell



Measured and simulated



Simu

The effect of the shell contour on performance was likely positive*, though complicated by differences in gas pressure

Parameter	N200715-001	N201021-002
Shell contour	No	Yes
Laser energy	785 kJ	804 kJ
Gas pressure	6.57 atm	5.46 atm
DD-n yield	1.1e12	1.1e12

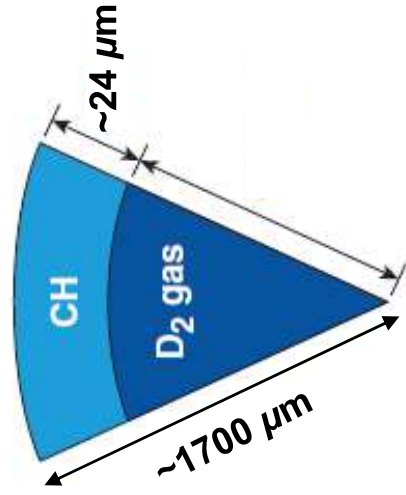
*Yield was the same, but this is likely attributed to gas pressure on the shot with the contour, which suggests a ~30-40% drop in yield

Recent experiments demonstrated control of symmetry and energy coupling through laser pointing and pulse shaping

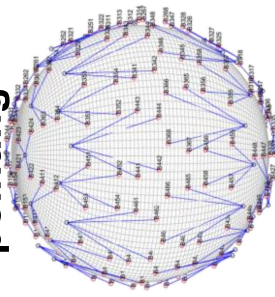
Exploding pusher

~700 kJ

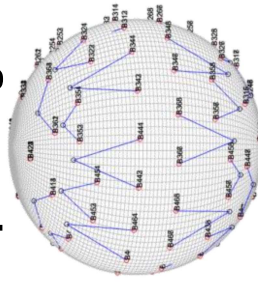
~280 TW



SAGE (old) pointing

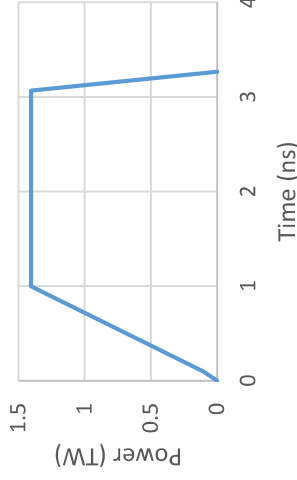


DRACO (new) pointing



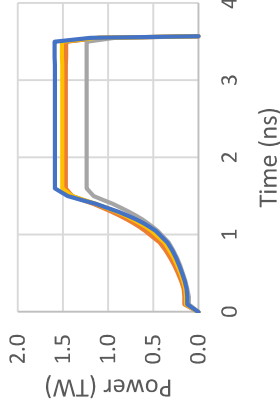
N201220-002

Per-Beam Laser Power



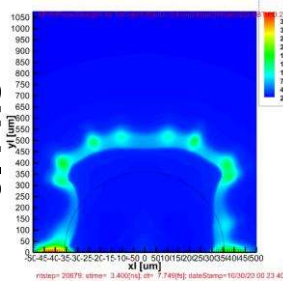
N201220-001

Per-Beam Laser Power



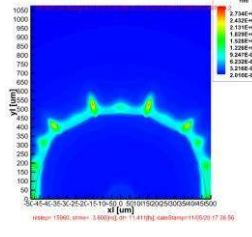
DRACO pre-shot

3.4 ns



DRACO pre-shot

3.8 ns

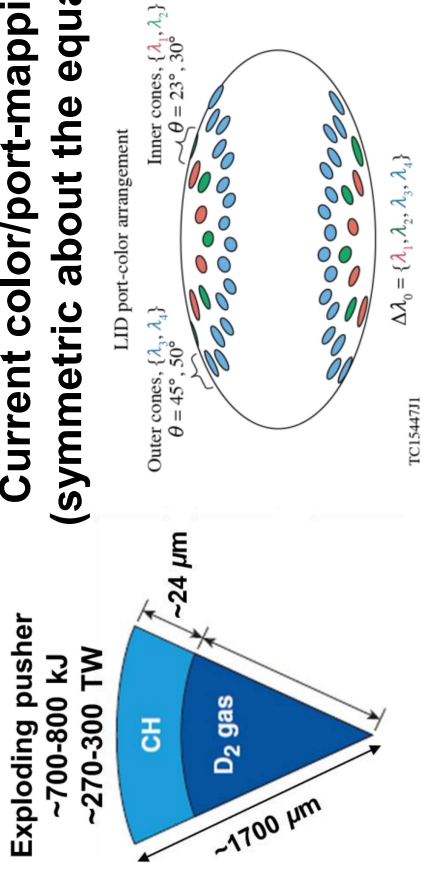


Design: J. Marozas

Yields were the same ~4.5e13; DRACO predicted 001 ~9e13, 002 ~5e13; shape improved translate to expected improvement in yield – post-shot simulations suggest laser imprint

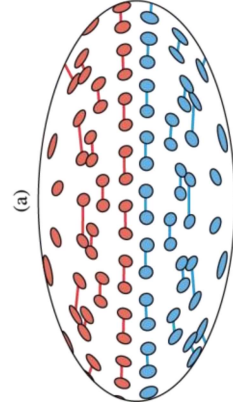
Upcoming experiments will test a new flexible color/port-mapping capability more effectively mitigate CBET in PDD using wavelength detuning

Current color/port-mapping (symmetric about the equator)

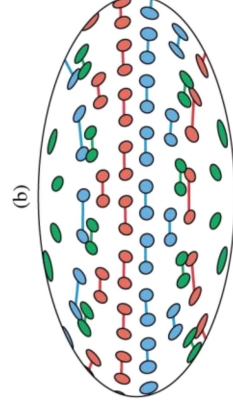


Options for flexible color/port-mapping

Bi-color hemispheric



Tri-color (example)



Indicates quad-split ports

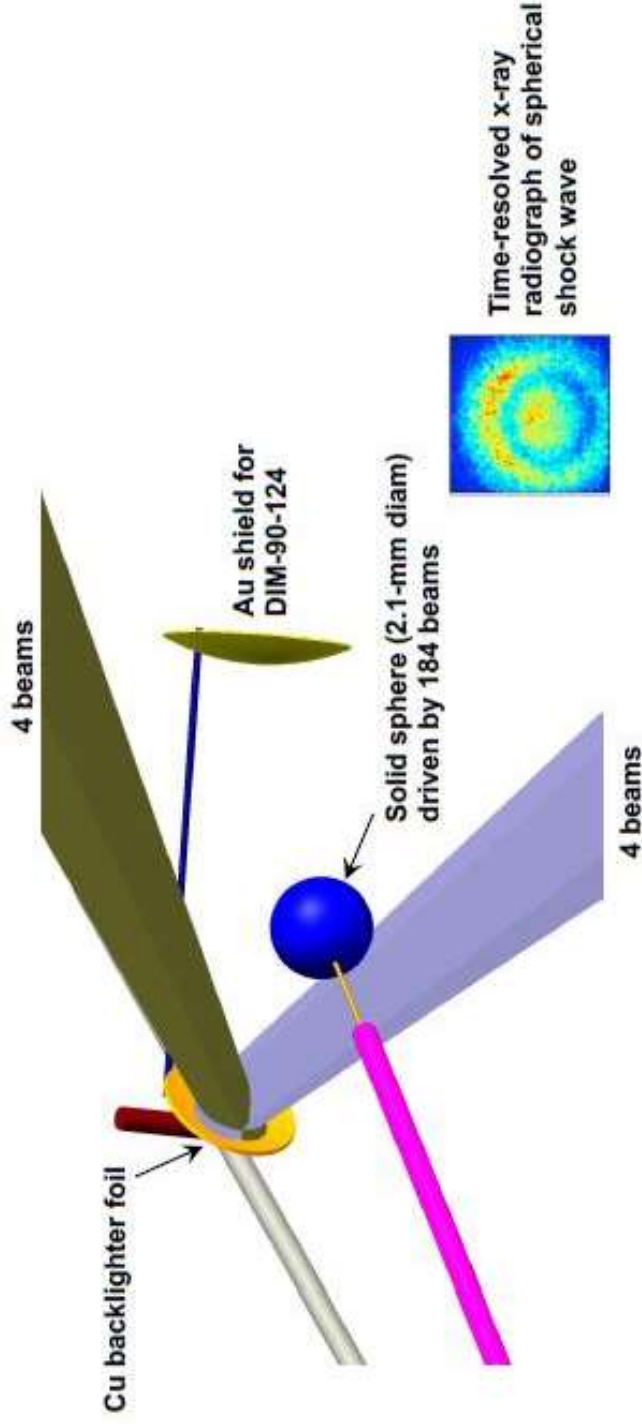
Design: J. Marozas (LLE)

fC2Pm Config	North → Eq	Eq → South	f_{abs} [
TriColor-BiColorHemi			
Bi-color	RRRR	BBBB	Pending
Tri-color Var#1	GGBB	RRGG	79.1
Tri-color Var#3	BBBG	RRGG	82.1

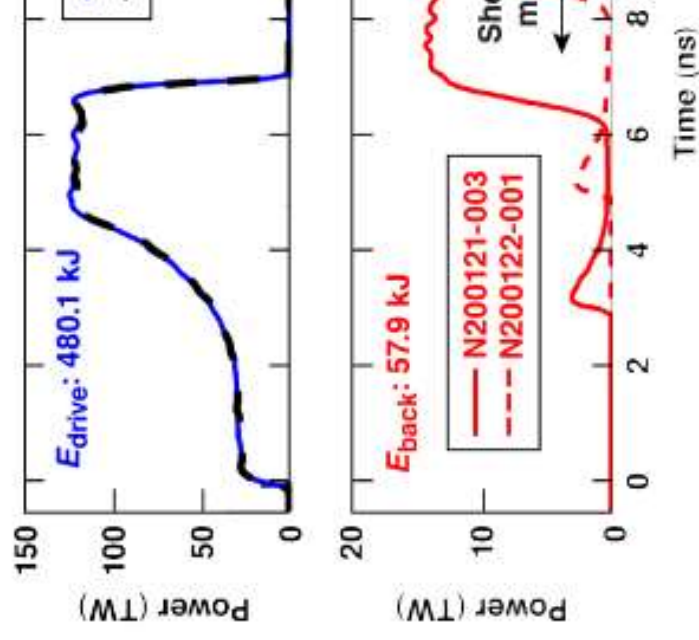
* $\Delta\lambda = \pm 4.5 \text{ \AA}$ (

Solid spheres are completely impervious to imprint and provide an isolated test of energy coupling and CBET modeling

Solid sphere radiography platform design

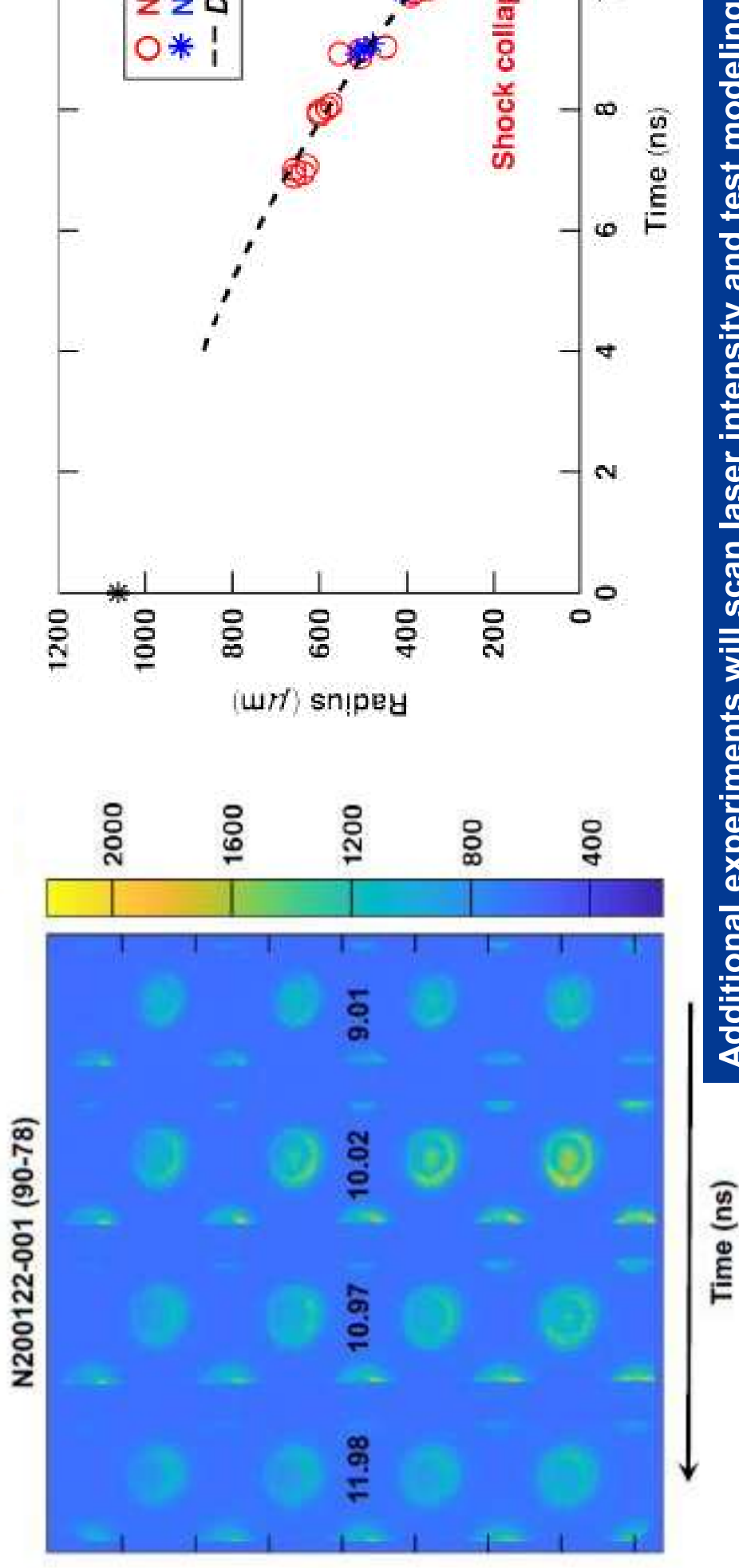


Laser driven
(8×10^{14} W/cm² intensity)



Experiment PI: W. Theobald
Designer: R. Bahukutumbi

Shock trajectory inferred from radiographs and x-ray flash time are in good agreement with 2D DRACO simulations

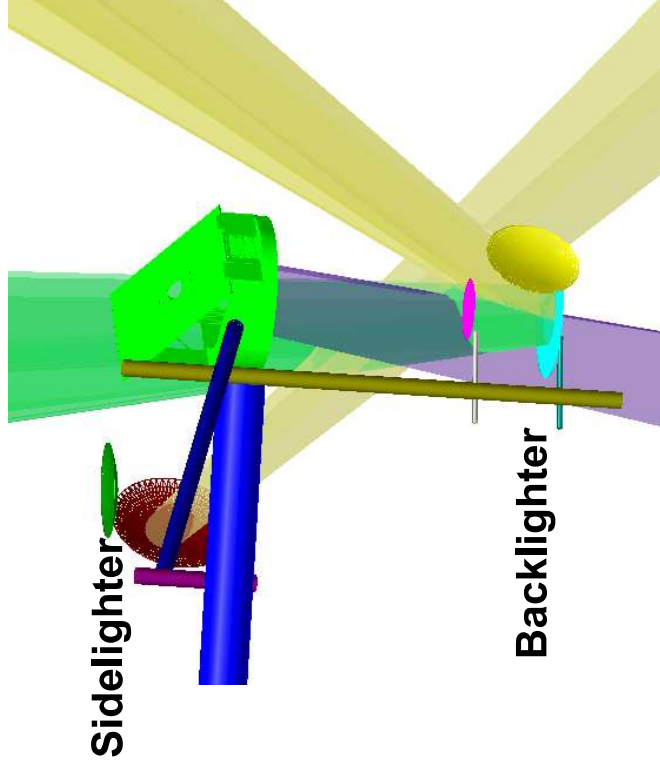


Additional experiments will scan laser intensity and test modeling scaled versions of these experiments are planned for

Experiment PI: W. Theobald
Designer: R. Bahukutumbi

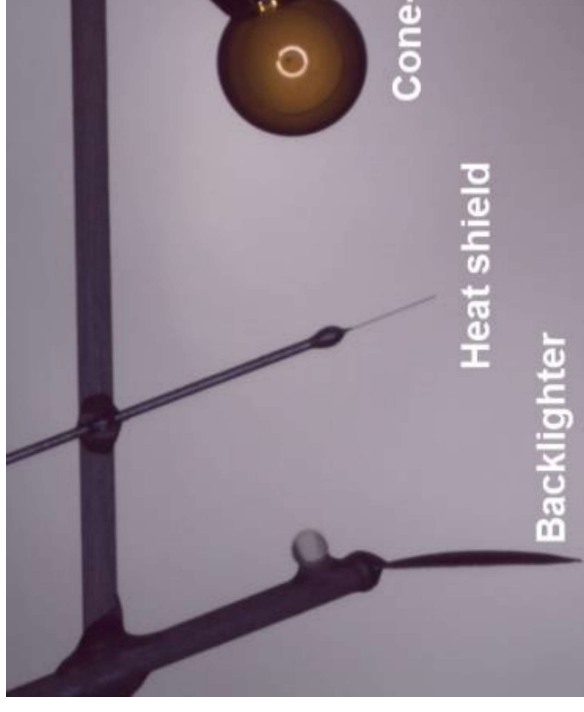
Laser imprint experiments are conducted on NIF to benchmark m and develop mitigation techniques

Planar Imprint Platform



Experimental PI: M. Hohenberger (LLNL), M. Rosenberg
Theory PI: A. Shvydky

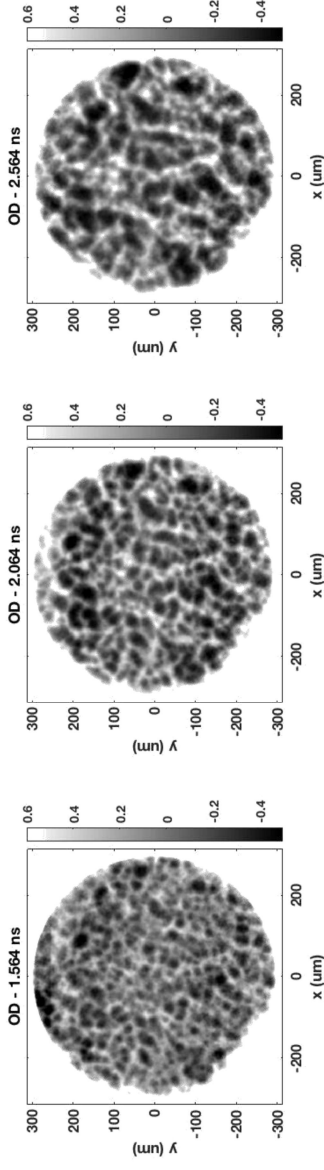
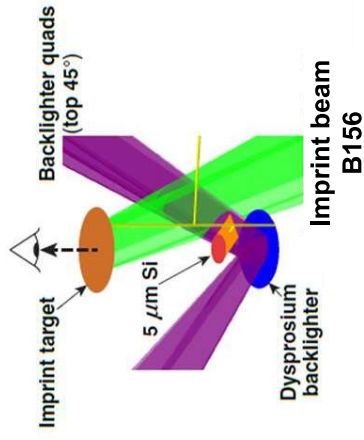
Spherical Imprint Platform



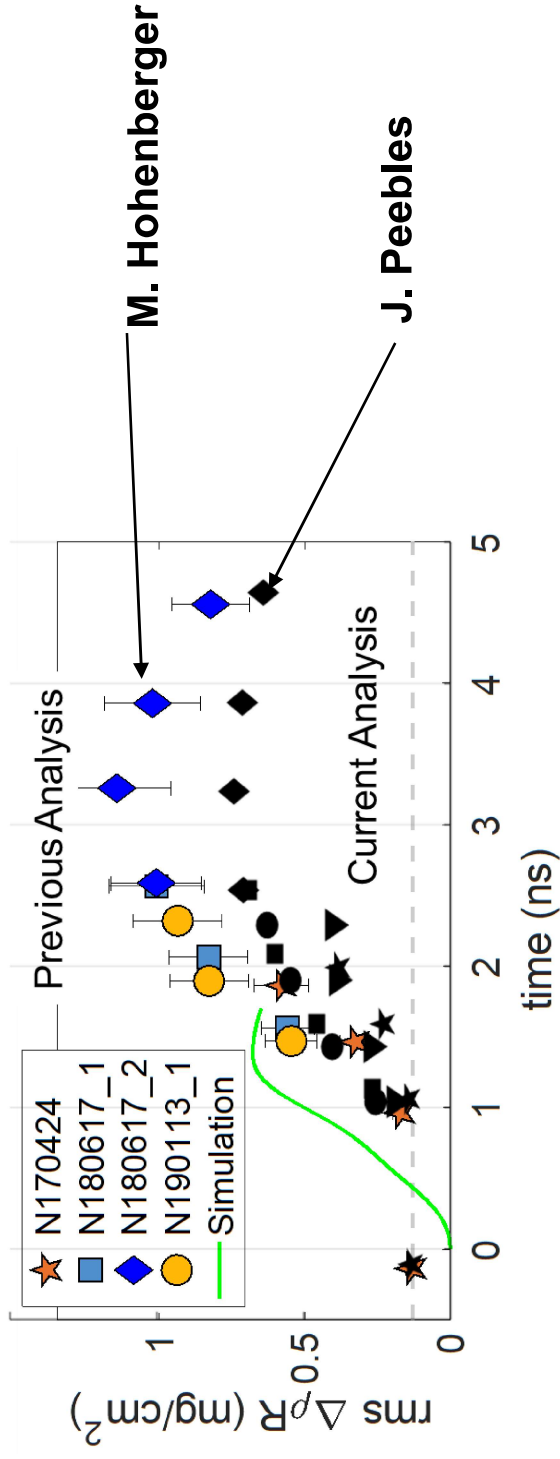
Experimental PI: M. Karasik (NIF)
Theory PI: A. Shvydky

Ultimate goal is to set requirements for laser and/or target solutions for imprint mitigation (how much/what is needed?)

Planar single-beam imprint experiments have produced excellent data that are in good agreement with HYDRA predictions and earlier/low energy experiments. Sidelighting was developed to validate modeling



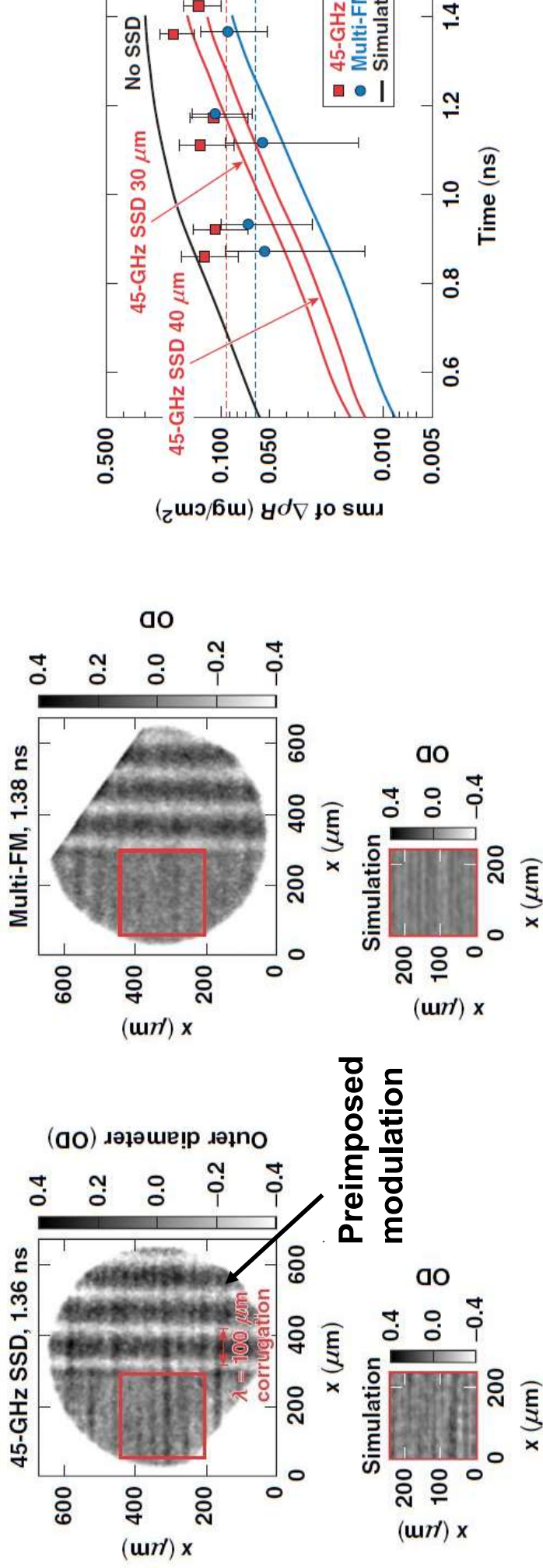
RT bubble morphology
imprint is visible
HYDRA predictions
and earlier/low energy
observed experiments



Analysis: M. Hohenberger (LLNL), J. Peebles, L. Antonelli (York)
 Simulations: A. Shvydky

Sidelighting was developed to validate modeling

Imprint mitigation using multi-FM SSD on 1 quad was demonstrated in planar geometry

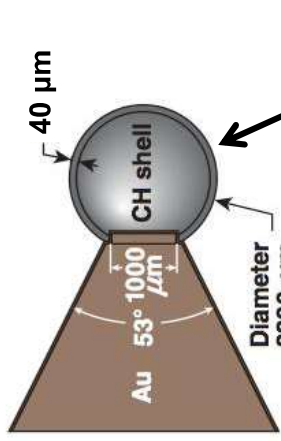


Preimposed modulation

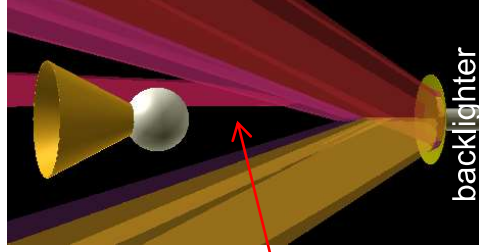
Experiment/analysis: M. Hohenberger (LLNL)
 Simulations: A. Shvydky

A next step for the imprint campaign may be the use of Fresnel zone plate imaging to achieve $\sim 5 \mu\text{m}$ resolution (vs. $15 \mu\text{m}$ now)

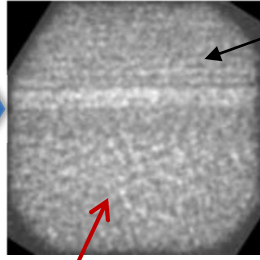
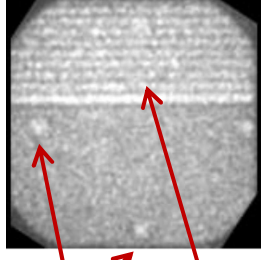
Cone-in-shell imprint experiments are assessing imprint from multiple overlapping beams and testing high-Z overcoats



early beam for soft x-ray prepulse to expand the coating 200J, at -10ns

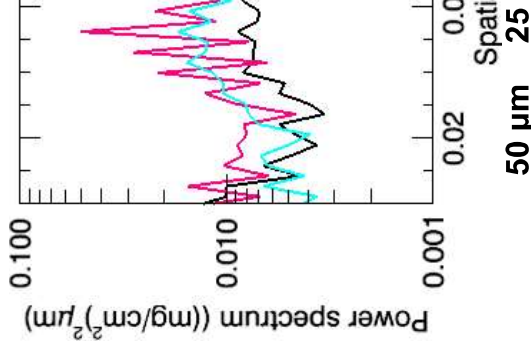
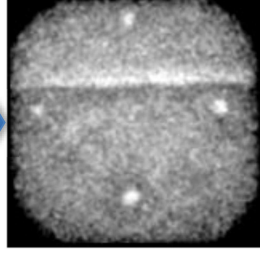
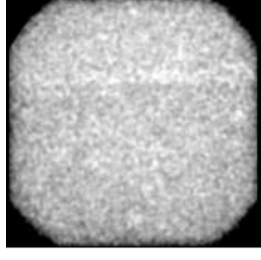


uncoated



Preimposed modulation

Au coated



Reduced growth of ~20-50 μm further analysis is o

Experiments/analysis: M. Karasik (NRL)
Design: A. Shvydky

Imprint mitigation with high-Z overcoat: Karasik et al. *Phys. Plasmas* 28, 032710 (2021)

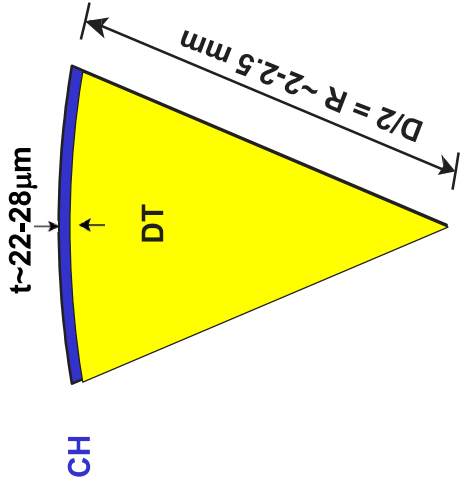
MJDD Campaign – what have we learned so far?

- Energy coupling/symmetry/CBET: symmetry and energy coupling can be controlled by wavelength detuning, shell contouring, and beam pointing
 - *Next steps*: test flexible color/port-mapping wavelength detuning, solid sphere experiments to benchmark modeling at different intensities/# of shocks
- LPI/preheat: SRS is the predominant hot electron source; preheat is close to tolerance levels for intensity $\sim 10^{15}$ W/cm²; Si layers are an effective mitigation strategy
 - *Next steps*: experiments to aid extrapolation of preheat in ignition-scale cryogenic experiments
- Laser imprint: we have two good platforms for studying imprint and imprint mitigation
 - *Next steps*: Benchmarking 3D calculations of imprint; higher imaging resolution; other target solutions for imprint mitigation?

Outline

- Laser direct-drive inertial confinement fusion
 - Scaling and motivation for NIF experiments
- MJDD experiments on NIF
 - LPI/hot electron preheat
 - Laser energy coupling/CBET/PDD symmetry
 - Laser imprint
- **Current high-yield PDD implosions and pathway to MJ yield**

Current high-yield PDD implosions (“exploding pushers”) are conducted with t LLE/LLNL Neutron Sources Working Group, and have routinely produced yield



High-yield PDD experiments

Shot	E [MJ]	P [TW]	Length [ns]	D [mm]	t [um]	SSD _{bw} [GHz]	P [at
N190227-001	1.1	400	3.7	4	25	90	8
N191020-001	1.6	430	4.6	4	29	90	8
N191027-003	1.3	440	3.9	4	23	90	8
N201128-001	1.6	470	5.1	5	24	45	6
N201221-002	1.8	480	5.0	5	22	45	6
N210213-001	1.8	490	4.6	5	23	45	6
Next shot	1.9	450	5.0	5	28	90	8

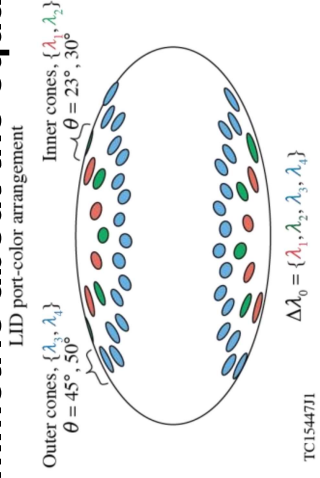
Experiments: C. Yeaman
 Designs: G. Kemp (LLNL)

Near-term goal: 2×10^{16} (~50 kJ)

Yeaman et al. *Nuclear Fusion* 61, 046031 (2021)

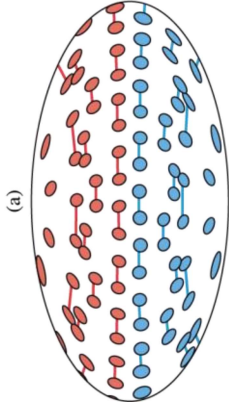
Near-term improvement in wavelength detuning (flexible color/port mapping in will facilitate better energy coupling and performance in PDD implosions

Current color/port-mapping (symmetric about the equator)

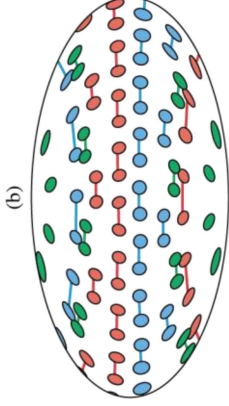


Options for flexible color/port-mapping

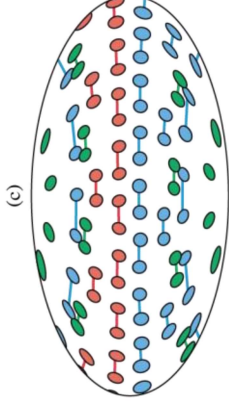
Bi-color hemispheric



Tri-color



Balanced tri-color



Design: J. Marozas

TC1543112

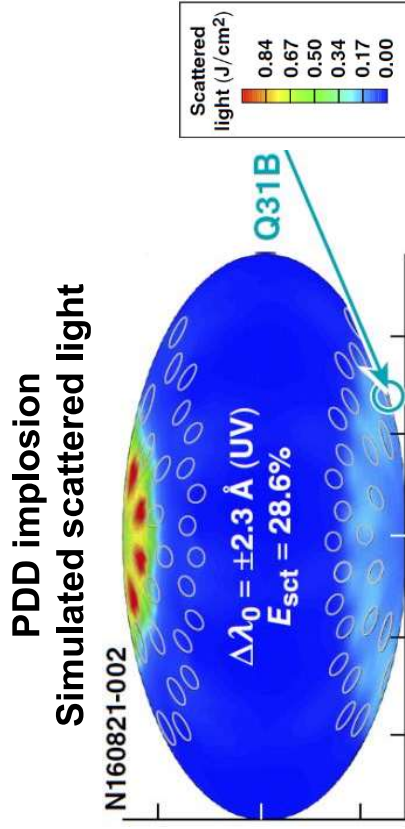
Flexible color/port-mapping is helpful, but significant detuning and yield will only be realized when wavelength detuning > ±3 Å UV (currently limited to ±2.3 Å UV)

Configuration	Pulse shape	Energy (MJ)	Total peak power (TW)	Abs frac
N190227 {reference sim.} => [N190227] _{ref}	ramp2FT	1.1	385	0.8
[Mod2] _{ref} and ±3 Å UV with fC2Pm	exp2FT	1.1	395	0.8
[Mod2] _{ref} and ±4.5 Å UV with fC2Pm	exp2FT	1.1	395	0.8
[Mod2] _{ref} and ±6 Å UV with fC2Pm	exp2FT	1.1	395	0.9

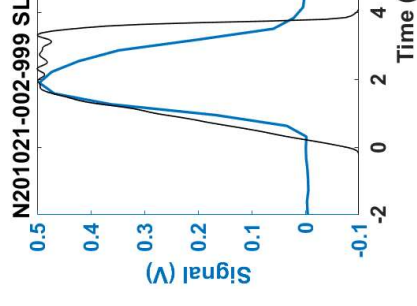
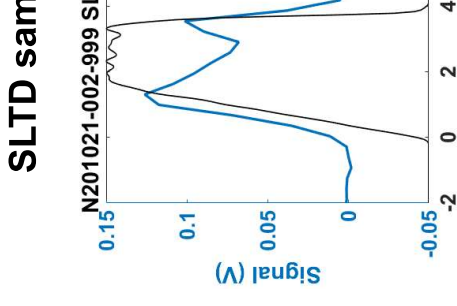
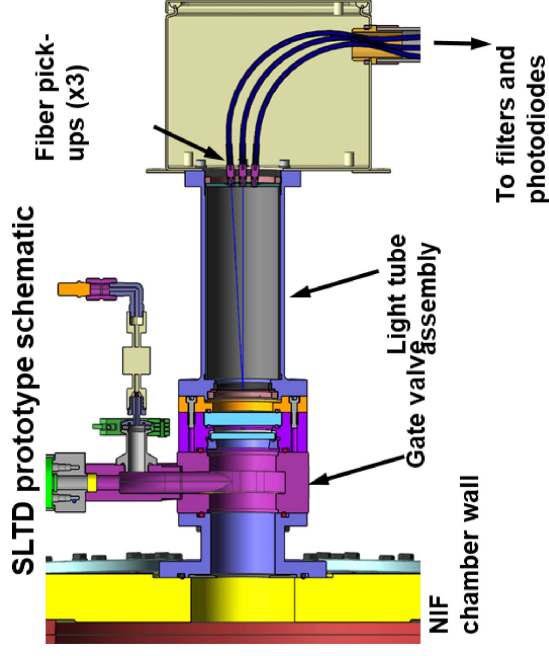
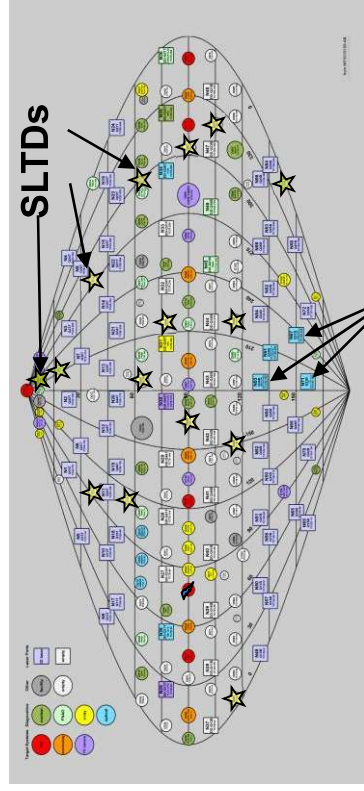
Proposed NIF facility improvements go hand-in-hand with steps to convergence and implosion performance (i.e. yield)

- 1) Current NIF
 - Low-convergence ($CR \leq 10$) implosions (“exploding pushers”)
 - Physics goal: optimize symmetry in current NIF, validate code
 - Yield: 50 kJ ($\sim 2e16$)
- 2) Flexible color/port-mapping, $\pm 6 \text{ \AA}$ UV, and PDD phase plates (\sim mid 2020s)
 - Low-convergence ($CR \leq 10$) implosions (“exploding pushers”)
 - Physics goal: improve symmetry, absorption $> 85\%$
 - Yield: 100 kJ ($\sim 4e16$)
- 3) PDD cryogenic target system, beam smoothing (\sim late 2020s)
 - Warm and cryogenic ablatively-driven implosions ($CR \sim 15-20$)
 - Physics goal: increased compression, alpha heating, burning
 - Yield: 300 kJ ($\sim 1e17$) to 1 MJ ($\sim 4e17$)

New diagnostics will contribute to improved understanding, e.g. of light diagnostics (SLTD) at a variety of angles



NIF chamber map



SLTD schedule

1 SLTD unit	April 2018
6 SLTD units	August 2019
11 SLTD units	January 2021
15 SLTD units	Late 2021

SLTD suite will constrain modeling of scattered light angular distribution and facilitate inference of total scattered light in direct-drive experiments

Rosenberg et al. *Rev. Sci. Instrum.* 92, 033511 (2021)

Current MJ-scale experiments lay the groundwork for high-yield direct-drive implosions on the NIF in the 2020's

- Extrapolation of direct-drive implosion performance from OMEGA to ignition scale on dedicated physics experiments at NIF scale
- The MegaJoule Direct Drive (MJDD) campaign on NIF includes three primary physics experiments:
 - 1) Hot electron preheat has been measured at close to tolerable levels, partially n using mid-Z layers, and shown to originate from stimulated Raman scattering
 - 2) Implosion symmetry and energy coupling (including cross-beam energy transfer) been controlled with wavelength detuning, contoured shells, and beam pointing
 - 3) Laser imprint experiments have produced x-ray radiography data to benchmark of imprint-related instabilities and partial mitigation with multi-FM SSD or high
- Current NIF capabilities limit high-performance polar-direct drive (PDD) implosion convergence exploding pushers

A staged set of proposed facility improvements will allow for increased convergence